

DE LA RECHERCHE À L'INDUSTRIE



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NUCLEAR FUEL CYCLES

6 – PARTITIONING, HOW ? *(an introduction to actinide separation processes)*

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Joliot Curie School, St Pierre d'Oleron, 22-27 September, 2019

An PARTITIONING, HOW ?

- 6-1 **ABOUT SEPARATION SCIENCE**
(some basic notions, application to An elements)
- 6-2 **PUREX**
(successive reprocessing operations)
- 6-3 **MINOR ACTINIDE SEPARATION**
(some basis for molecular and process design)
- 6-4 **OTHER FUELS PROCESSING**
(thorium fuels, ...)
- 6-5 **PYRO PROCESSES**
(an introduction to...)

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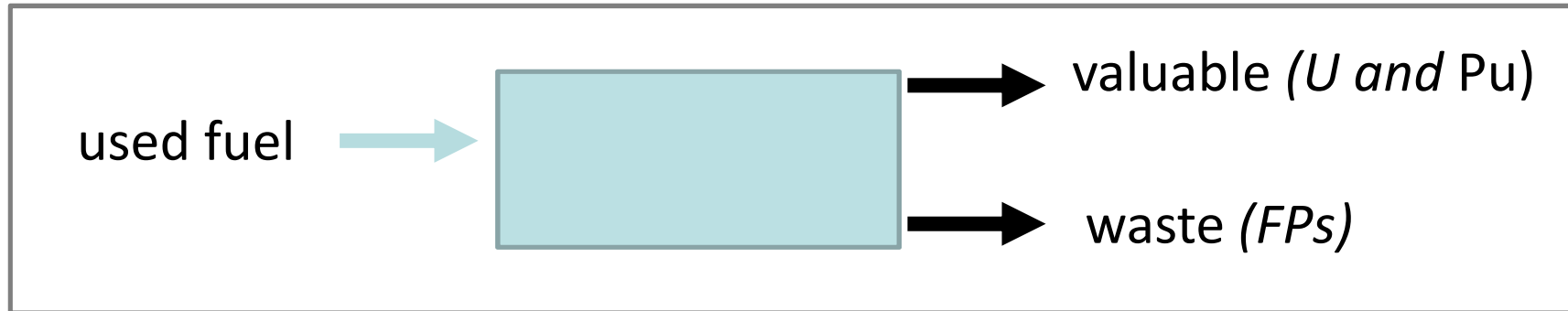
NUCLEAR FUEL CYCLES

6-1 SEPARATION : BASIC NOTIONS

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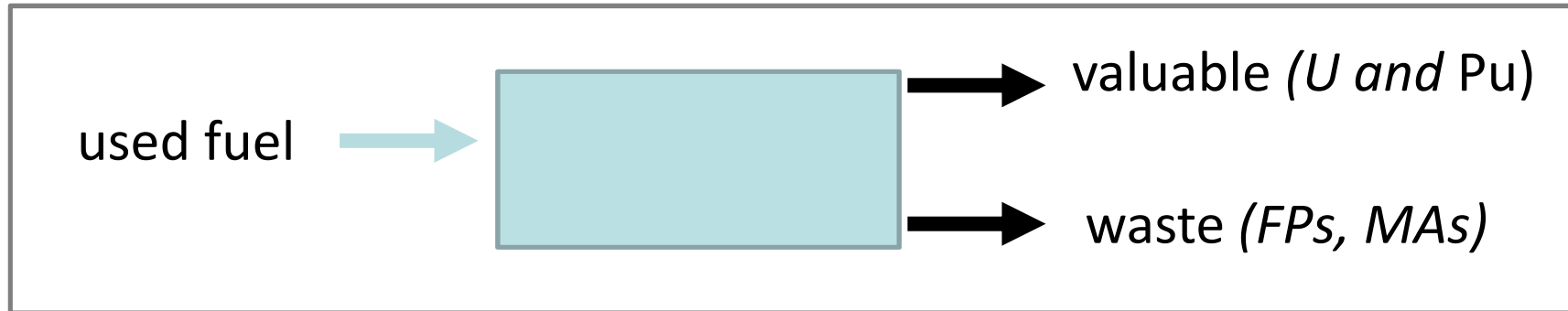


SPECIFICATIONS:

- high recovery yields for valuable elements
- very high purification factors for recovered valuable elements

CONSTRAINTS :

- (cost)
- low amount of secondary radwaste
- confinement (avoid dispersion)
- protection (low doses)
- criticality risk management
- radiolysis effects mitigation



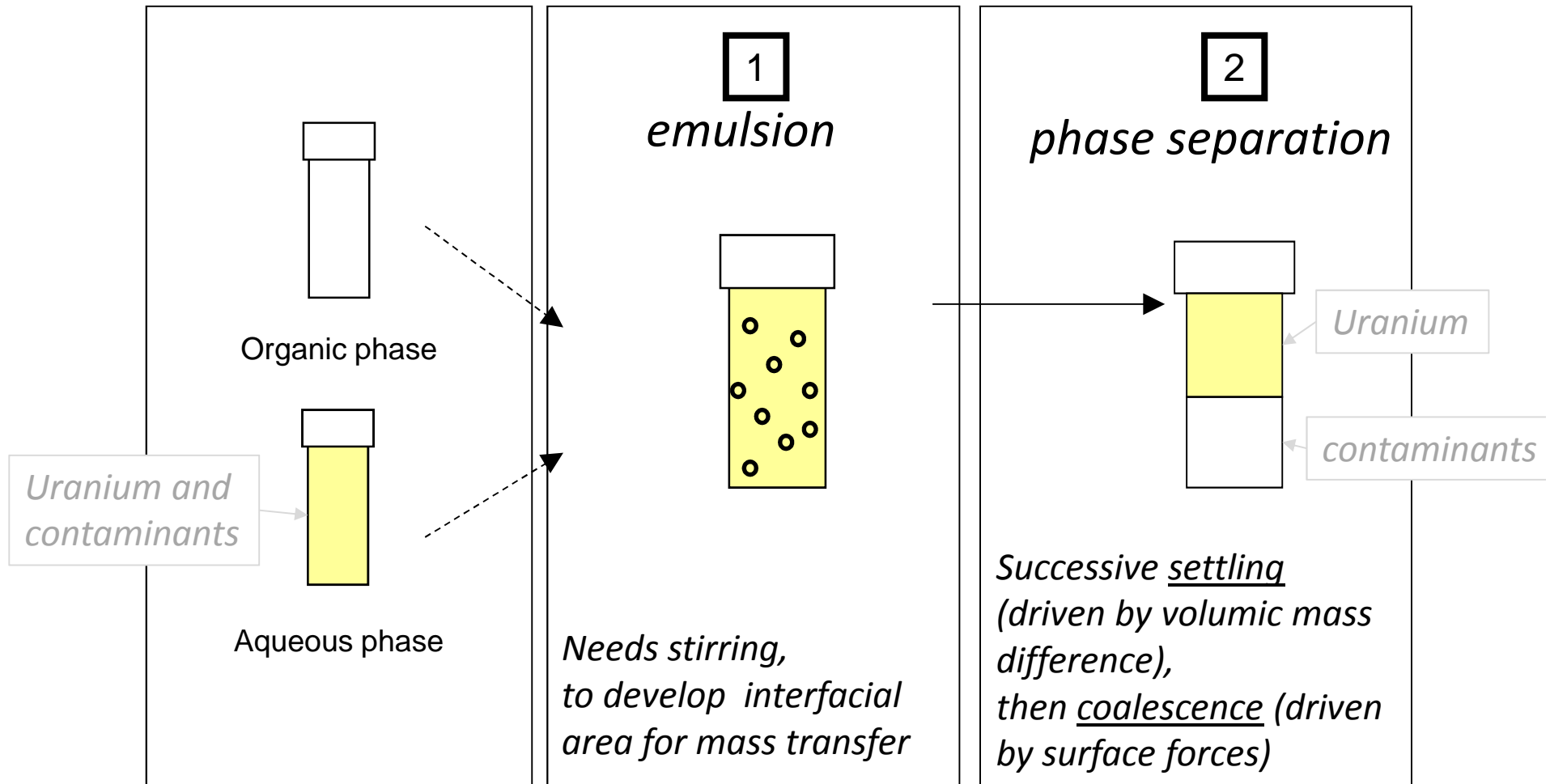
HOW TO PROCEED ?: *take advantage of differences*

- Physical properties (*mass, electric, magnetic...*)
- Chemical properties (*interactions with reagents,...*)

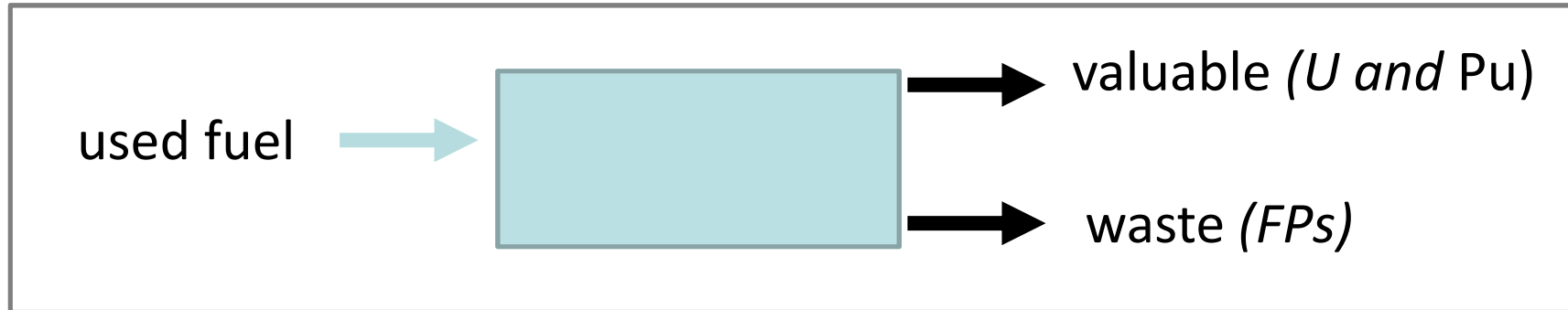
MAIN PROCESSES :

- "Aqueous processes" (*solvent extraction processes*)
[up to industrial scale]
- "Dry processes" (*electrorefining pyroprocesses, selective volatility*)
[laboratory scale]

SOLVENT EXTRACTION



SOLVENT EXTRACTION EFFICIENCY (global approach)



$$\mathbf{M \text{ recovery yield}} = \frac{\text{M amount in organic phase after extraction step}}{\text{M initial amount in aqueous phase}}$$

M1 purification (decontamination) factor vs. M2:

$$\mathbf{DF} = \frac{[M2]/[M1] \text{ in initial aqueous solution}}{[M2]/[M1] \text{ in recovered (organic) solution}}$$

SOLVENT EXTRACTION EFFICIENCY (elementary approach)

For each element M , present in initial aqueous solution:

$$D_M \text{ (or } K_D) = \frac{[\overline{M}]_{org}}{[M]_{aq}} \quad D_M \text{ adimensional}$$

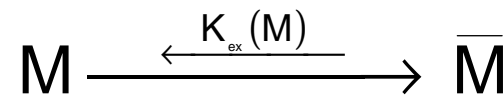
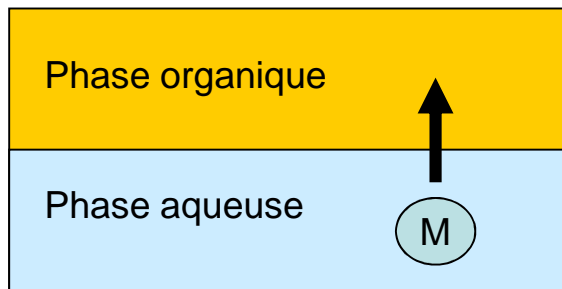
$$D_M = \frac{[\overline{M}] + [\overline{\text{complex 1}}] + [\overline{\text{complex 2}}] + \dots}{[M] + [\text{complex 1}] + [\text{complex 2}] + \dots}$$

For two elements, M_1 and M_2 to be separated :

$$SF_{M_1/M_2} = \frac{D_{M_1}}{D_{M_2}}$$

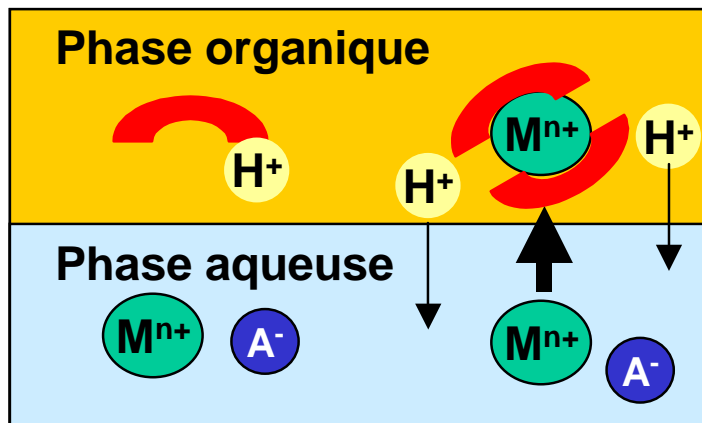
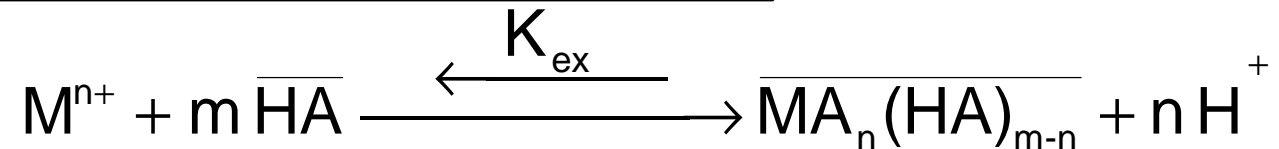
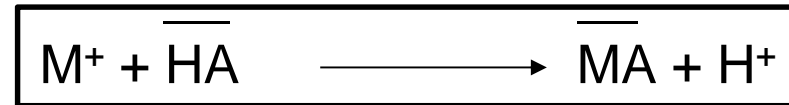
- « PASSIVE EXTRACTION »,

(solvation of a lipophilic compound in the organic phase)



- *Iodine in the system: water/CCl₄*

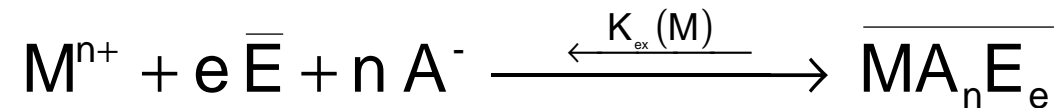
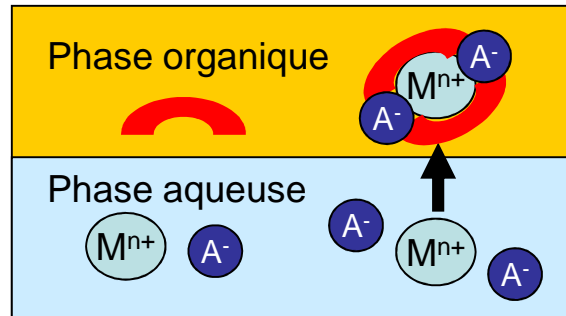
- **CATION EXCHANGE**



$$K_{ex} = \frac{[\overline{MA_n (HA)_{m-n}}][H^+]^n}{[M^{n+}][\overline{HA}]^m} = \frac{D_M \cdot [H^+]^n}{[\overline{HA}]^m}$$

- Am^{3+} by HDEHP (diethylhexyl phosphoric acid)

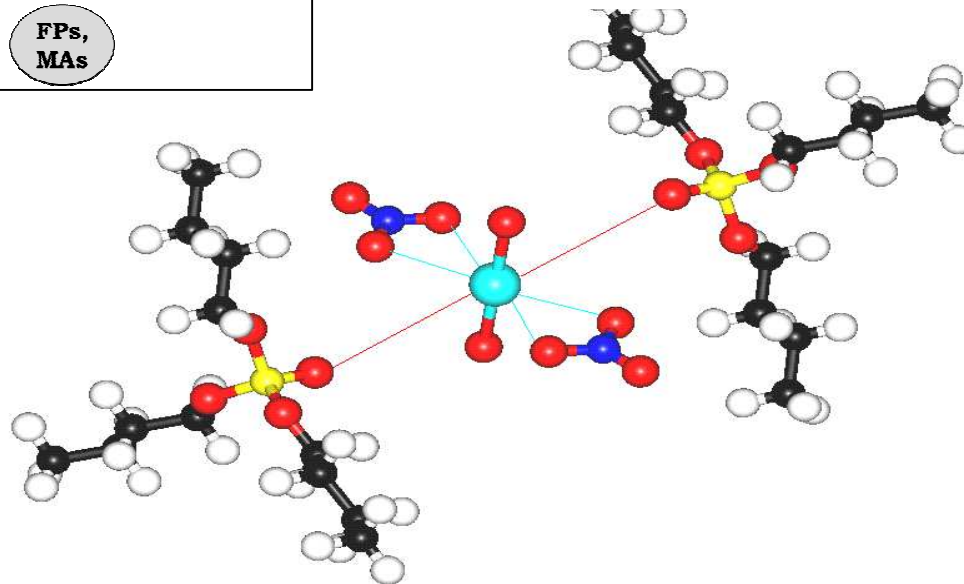
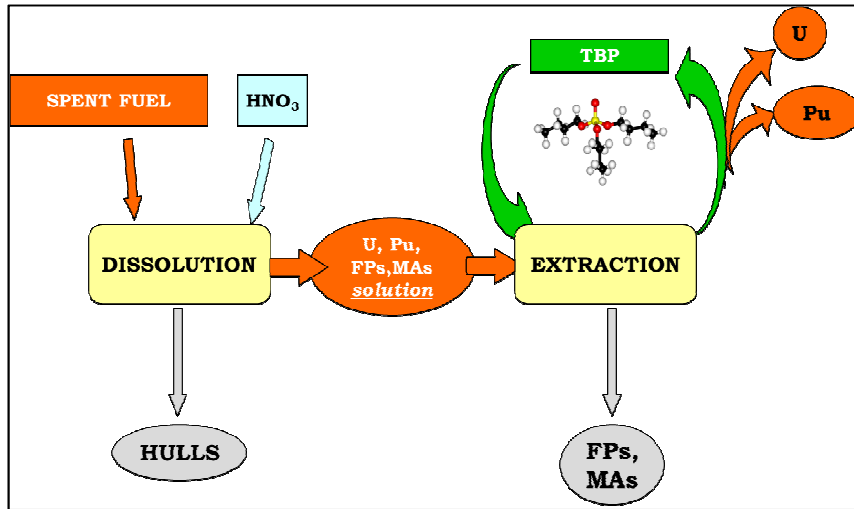
- **SOLVATION OF A NEUTRAL COMPLEX** :
(acid or metallic salt in the initial aqueous solution)



$$K_{ex}(M) = \frac{[\overline{MA_nE_e}]}{[M^{n+}] \cdot [\bar{E}]^e \cdot [A^-]^n} = \frac{D_M}{[\bar{E}]^e \cdot [A^-]^n}$$

- *Uranyl nitrate by TBP (tri-butyl phosphate)*

THE PUREX PROCESS



UO_2^{2+} EXTRACTION by TBP



- «salting out effect» : if $[NO_3^-] \nearrow$, then $D \nearrow$
- «saturation effect» : $[\overline{TBP}]$ defines $[\overline{U}]_{\max}$
(if $\overline{TBP} = 1.1 \text{ M}$, $[\overline{U}]_{\max} = 0,55 \text{ mol/l} \# 120 \text{ g/l}$)
- «compétition effect» : U, Pu, H^+ « candidates » for extraction
- «température effect» : if $T \nearrow$, then $D \searrow$

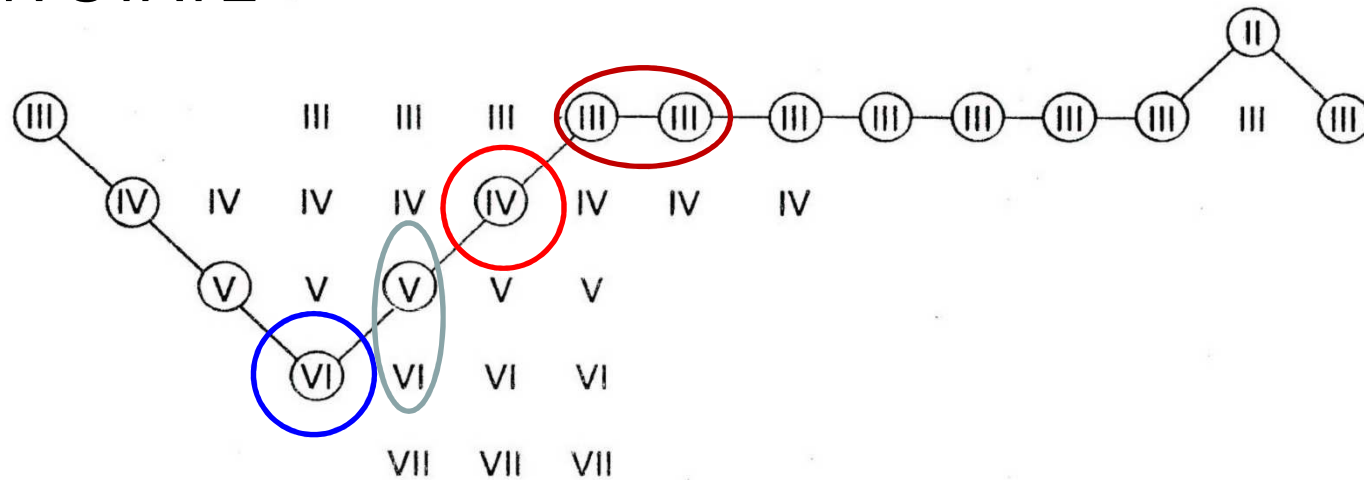
⇒ **potentially quantitative extraction**

⇒ **possible reversibility**

| | | | | | | | | | | | | | | | |
|--------|----|----|----|----|----|----|----|----|----|----|----|-----|-----|-----|-----|
| Z | 89 | 90 | 91 | 92 | 93 | 94 | 95 | 96 | 97 | 98 | 99 | 100 | 101 | 102 | 103 |
| SYMBOL | Ac | Th | Pa | U | Np | Pu | Am | Cm | Bk | Cf | Es | Fm | Md | No | Lr |



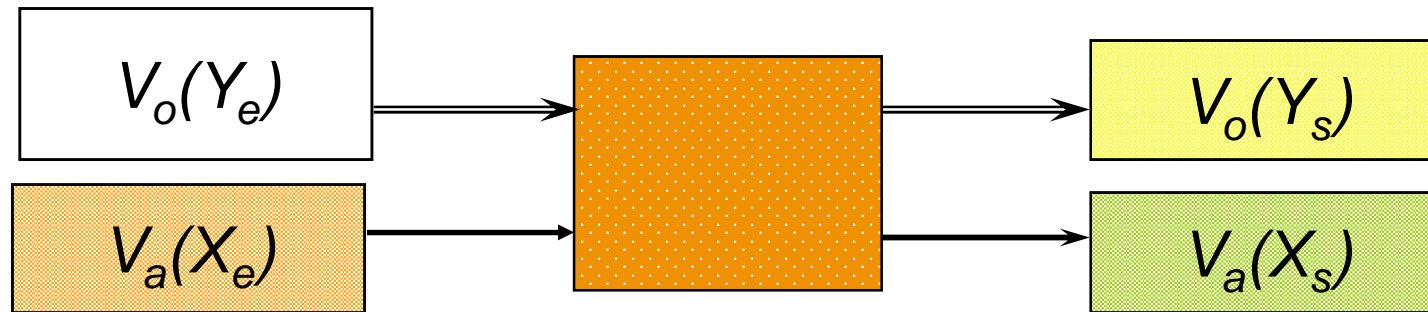
OXIDATION STATE



| Element | SPECIE | $D_{M^{n+}}$ |
|---------|----------------------------|--------------------------|
| U | UO_2^{2+} | 25 |
| Pu | Pu^{4+} | 11 |
| Np | NpO_2^{2+} (NpO_2^+) | 10 ($2 \cdot 10^{-2}$) |
| Am | Am^{3+} | $< 10^{-2}$ |
| Ln | Ln^{3+} | $< 10^{-2}$ |

$$D_{M^{n+}} = \frac{[M^{n+}]_{org}}{[M^{n+}]_{aq}}$$

25°C, [TBP] = 1,1 M in dodecane, [HNO₃] = 3 M



DEFINITION :

- Outlet phases at thermodynamical equilibrium
- Outlet phases perfectly separated

BASIC RELATIONS :

- équilibre : $Y_s = D.X_s$
- mass balance : $V_a(X_e - X_s) = V_o(Y_s - Y_e)$

(Hypothesis : organic phase without solute before contact)

- **équilibre:** $Y_s = D.X_s$
- **Mass balance:** $X_e = (V_a.X_s + V_o.D.X_s) / V_a$ (batch)
 $X_e = (A.X_s + O.D.X_s) / A$ (continuous)

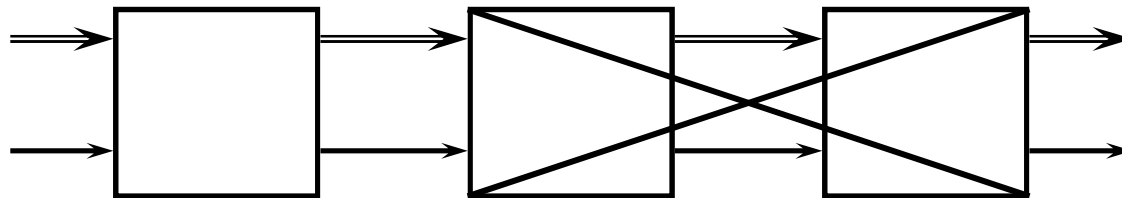
$$X_s = \frac{X_e}{1 + E} \quad \text{avec} \quad E = \frac{V_o}{V_a} \cdot D = \frac{O}{A} \cdot D$$

(E "extraction factor")

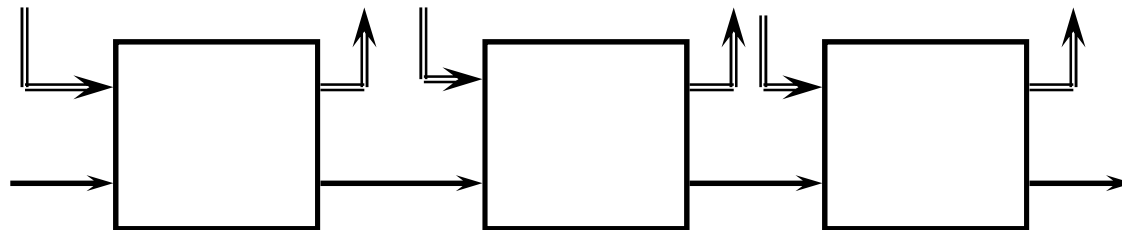
to get high E value $\begin{cases} \frac{V_o}{V_a} & \text{ou} & \frac{O}{A} & \text{high} \\ D & \text{high} \end{cases}$

LIQUID / LIQUID EXTRACTION : *CONTINUOUS & REPEATED PROCESSES*

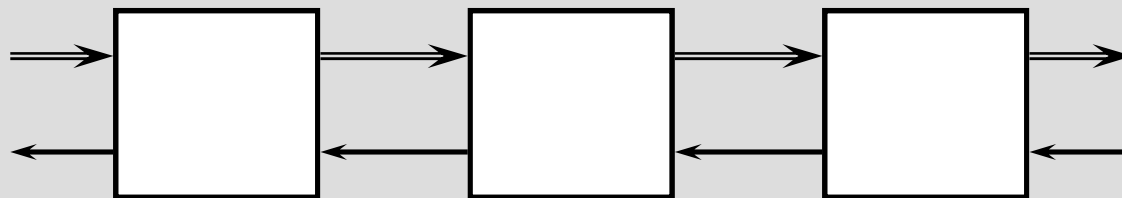
Co-flow ?



Possible flow-sheet :



Cross-flow: the way for high separation yields



EFFECT OF CUMULATIVE CROSS-FLOW STAGES : KREMSER RELATION

$$\frac{X_{\text{inlet}}}{\mathcal{E}_{\text{residue}}} = \frac{E^{n+1} - 1}{E - 1}$$

with

$$E = \frac{O}{A} \cdot D$$

A aqueous flowrate

O extractant flowrate

D distribution coeff.

Ex: $D=10$, $O=A$, $n= 1$, residue fraction = $1/10$

2

$1/100$

3

$1/1000$

...

....

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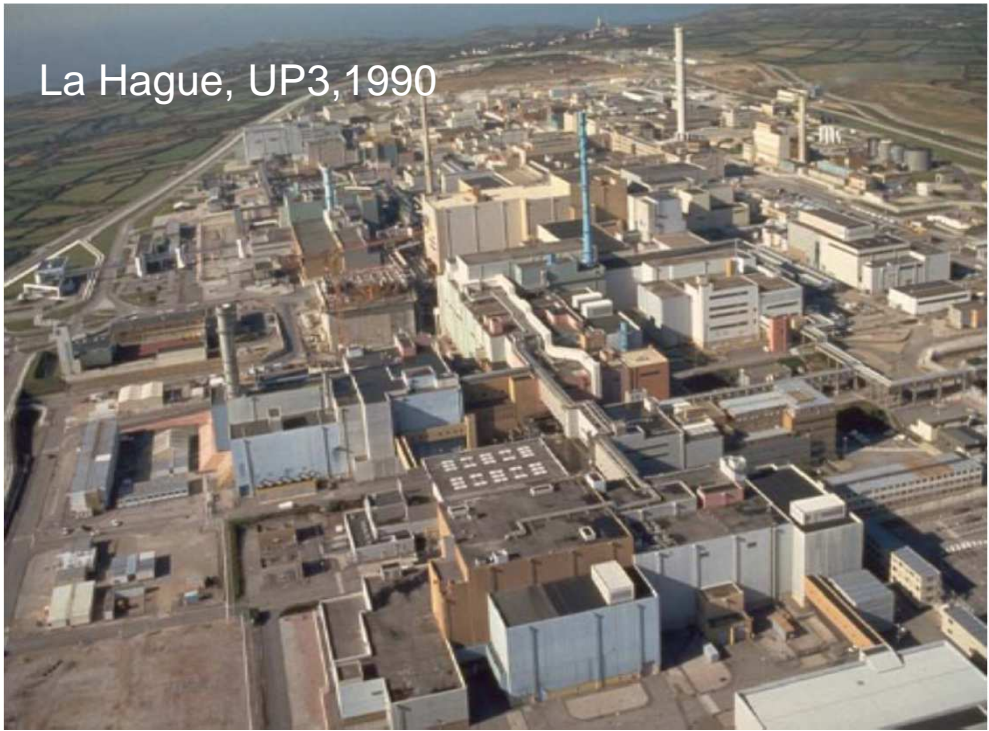
NUCLEAR FUEL CYCLES

6 -2 THE PUREX PROCESS

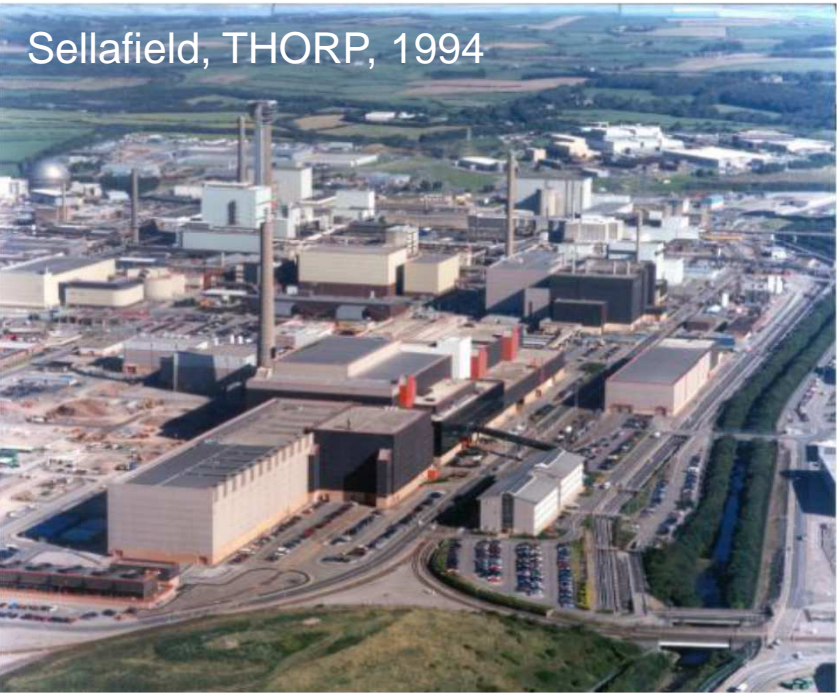
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La Hague, UP3, 1990

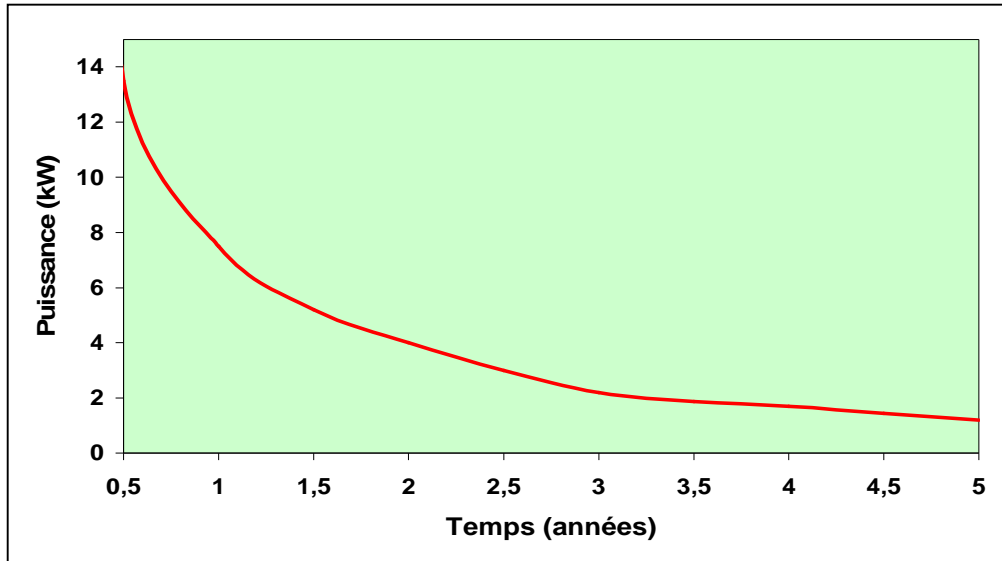


Sellafield, THORP, 1994

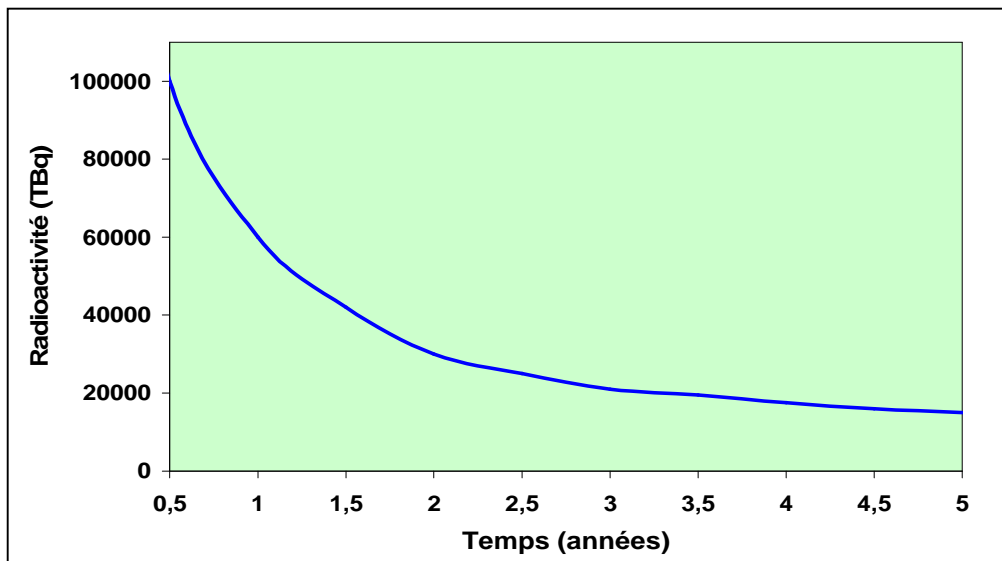


Rokkashomura

PUREX PROCESS: CURRENT INDUSTRIAL OPERATION (1)



➤ Residual heat
(assembly)



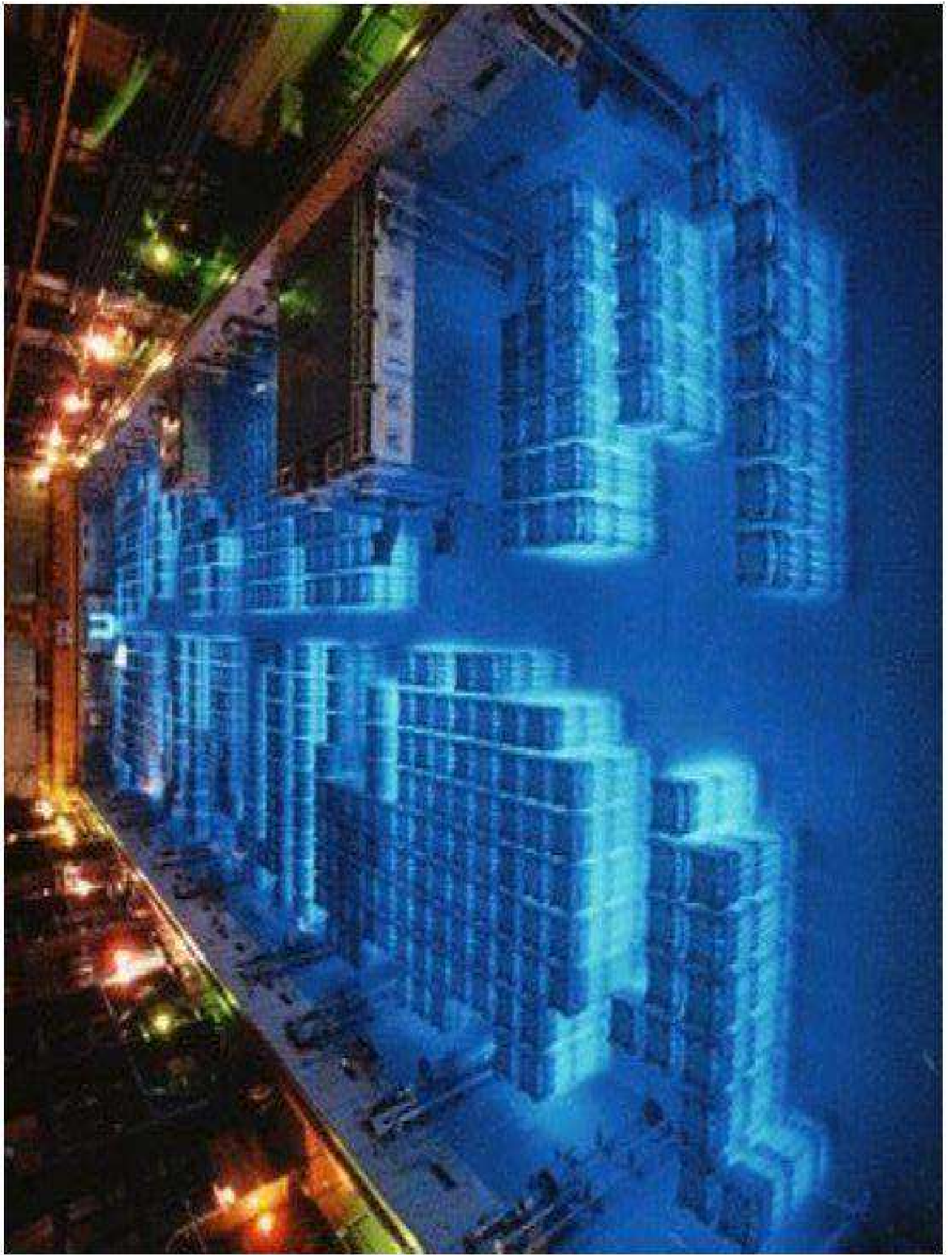
➤ Radioactivity
(assembly)



USED FUELS TRANSPORTATION

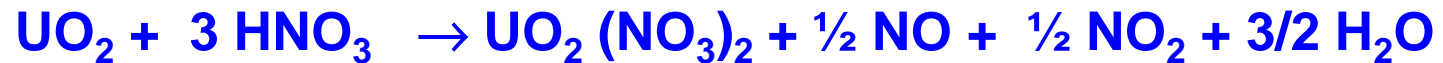






PUREX PROCESS: CURRENT INDUSTRIAL OPERATION (2)

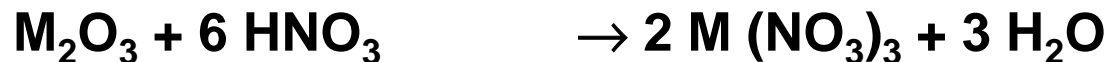
✓ UO₂ DISSOLUTION : $\Delta H = - 420 \text{ J/kg}$



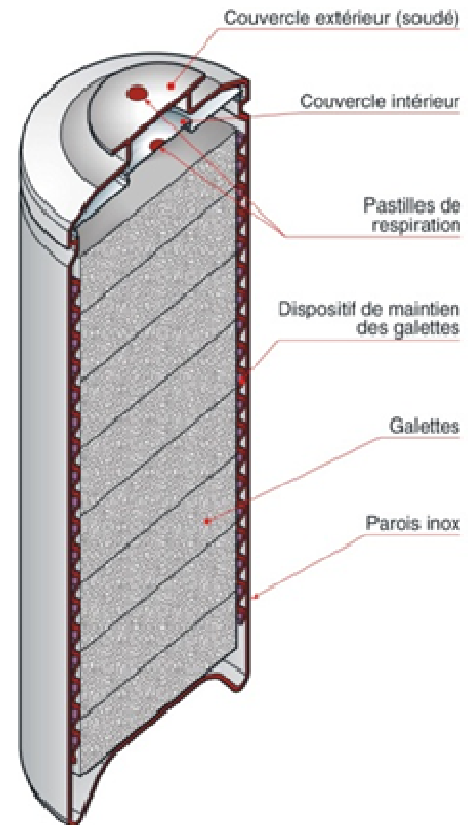
✓ PuO₂ DISSOLUTION :



✓ PF/TU (III) DISSOLUTION :

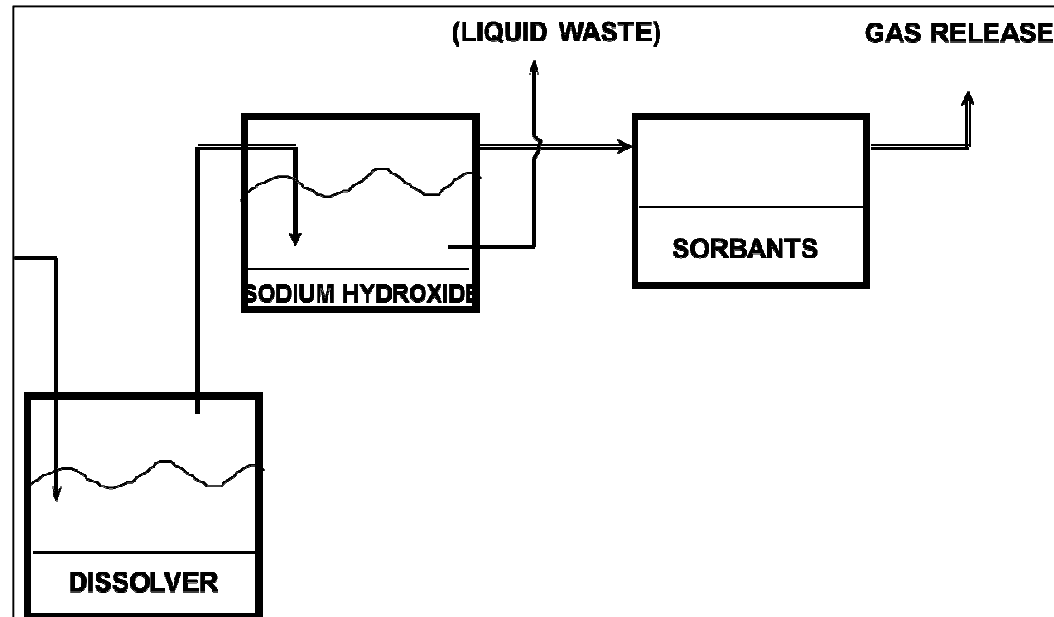


CLADDING MATERIAL



PUREX PROCESS: CURRENT INDUSTRIAL OPERATION (3)

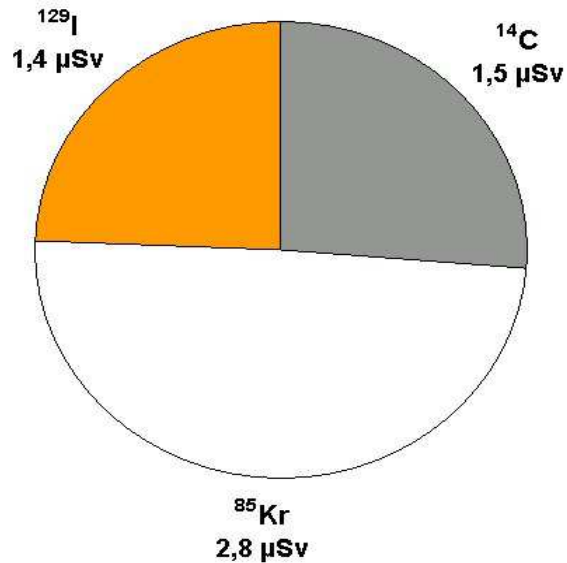
- IODINE (I_2) [^{129}I]
- CARBONE (CO_2) [^{14}C]
- TRITIUM (HTO) [3H]
- KRYPTON (Kr) [^{85}Kr]



IMPACT REJETS LA HAGUE

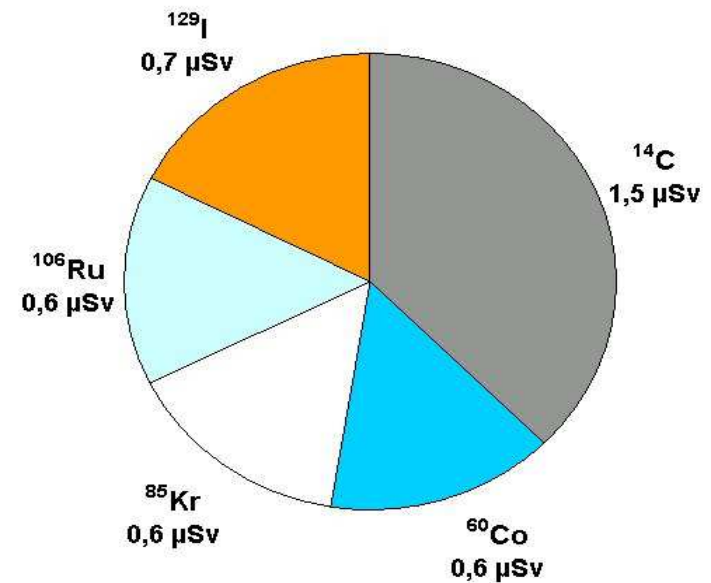
(groupe radioecologie Nord Cotentin, 1996)

Dose efficace totale = 6 μ Sv



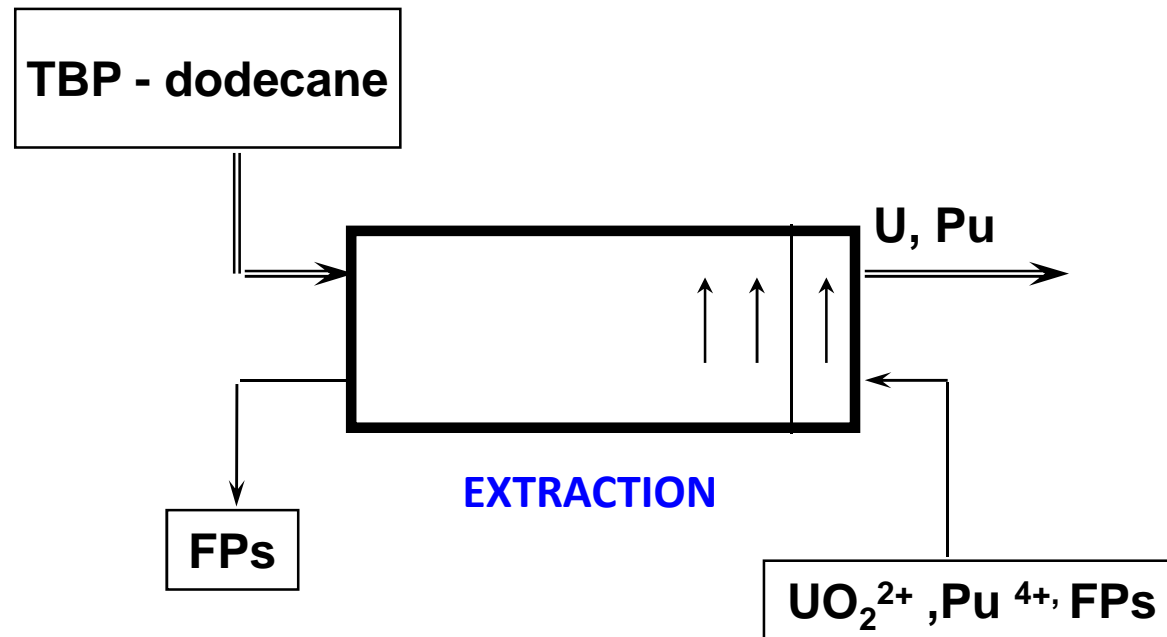
"Adultes de Digulleville" (groupe de référence associé aux rejets atmosphériques), 1996

Dose efficace totale = 5 μ Sv



"Pêcheurs de Goury" (groupe de référence associé aux rejets liquides en mer), 1996

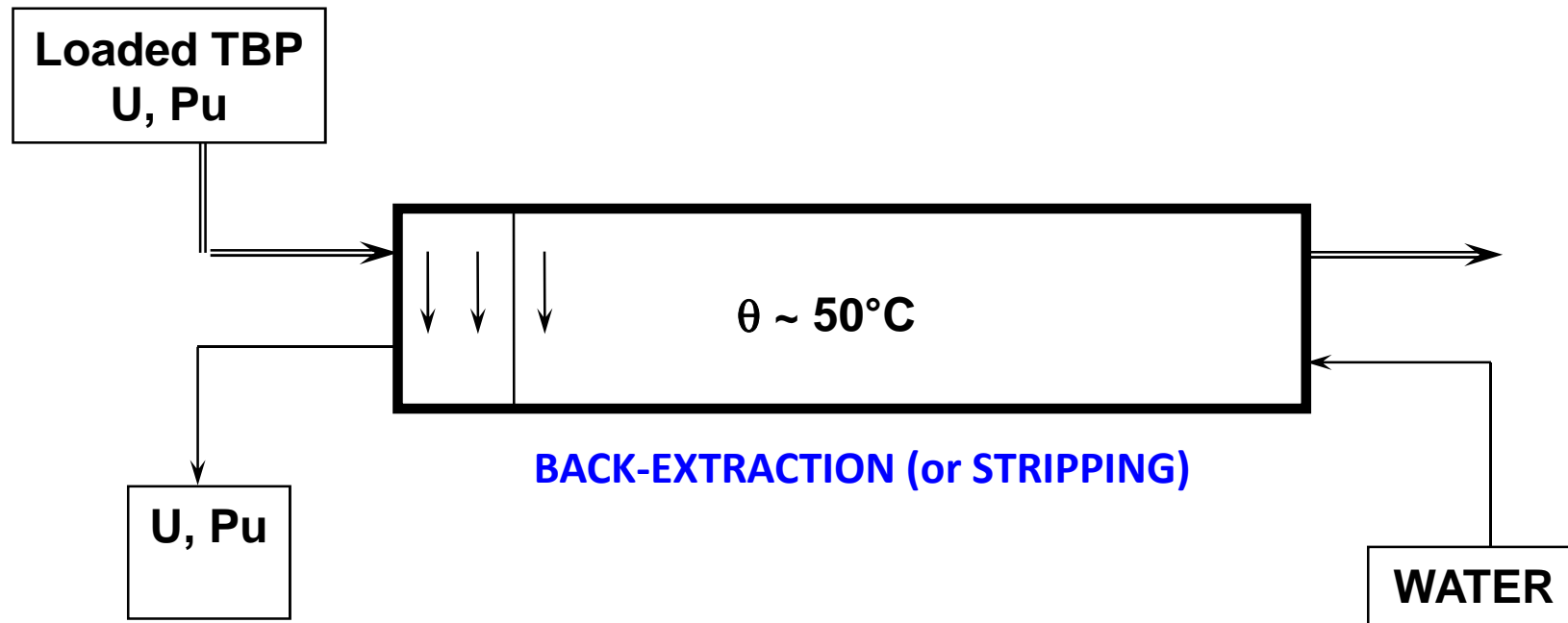
PUREX PROCESS: CURRENT INDUSTRIAL OPERATION (4)



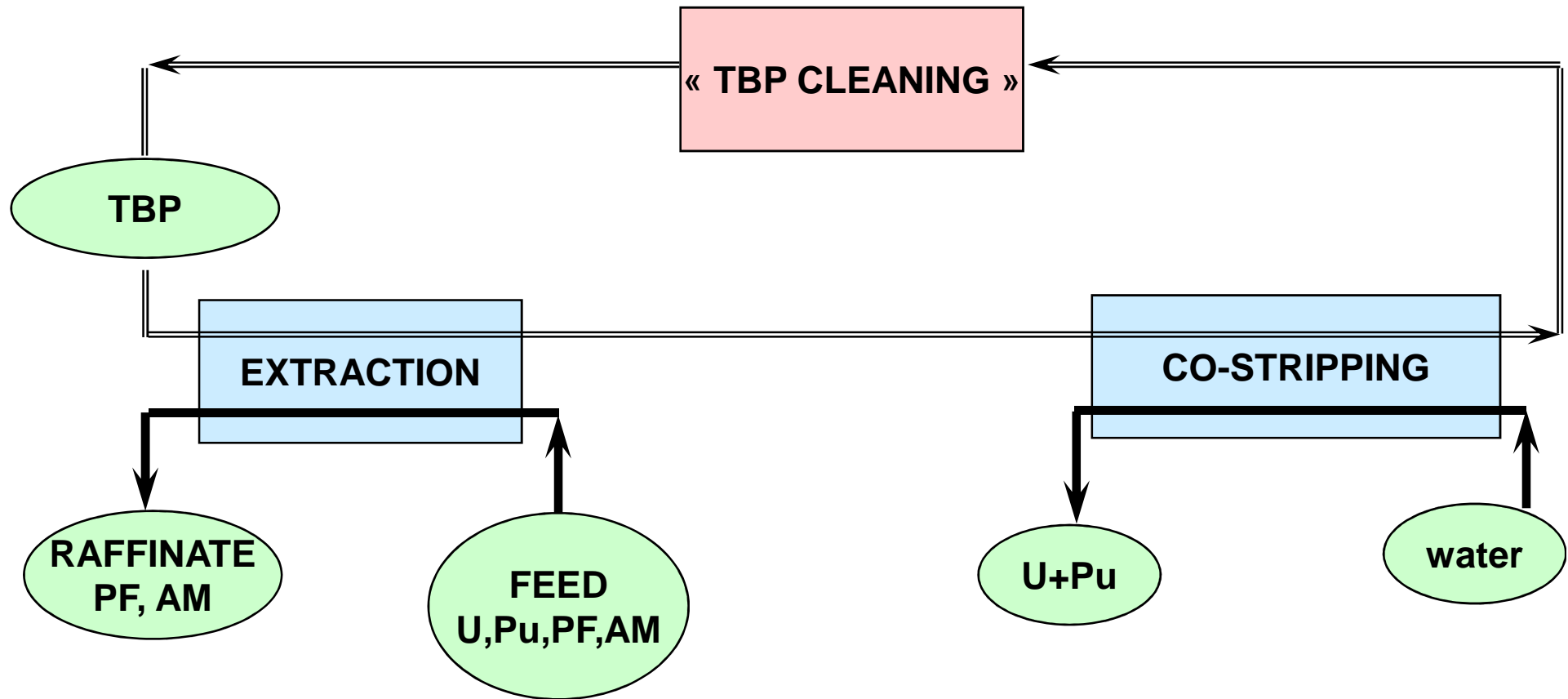
OPERATING CONDITIONS : $[\text{HNO}_3]$ rather high
 $[\text{U}]_{\text{org}}$ rather high



PUREX PROCESS: CURRENT INDUSTRIAL OPERATION (5)

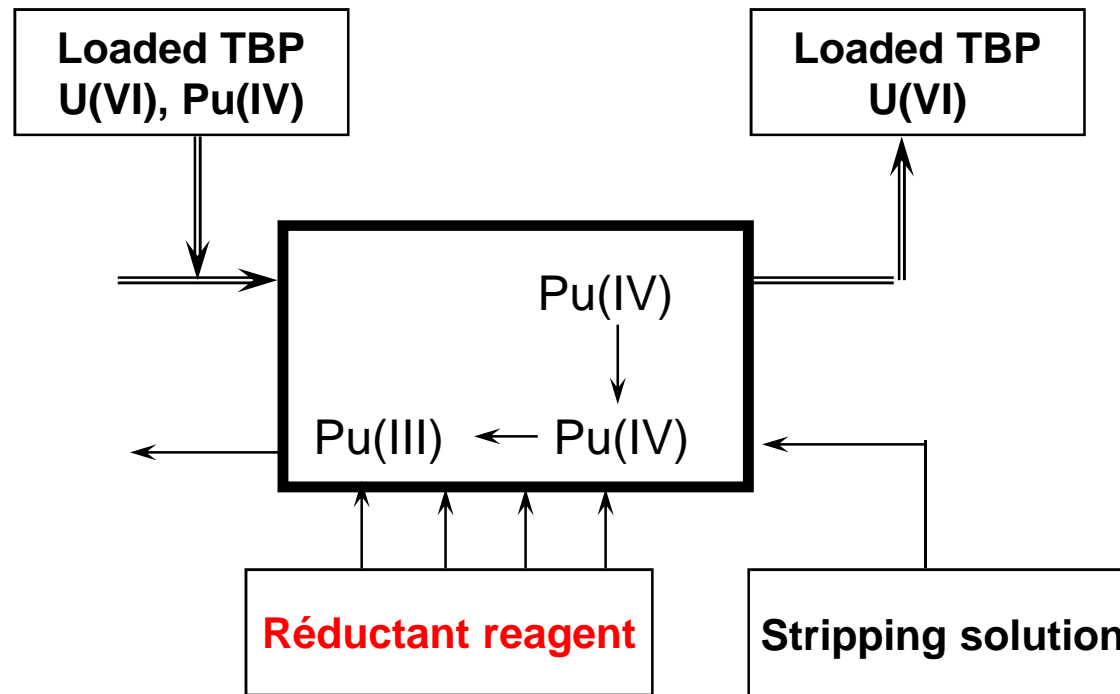


PUREX PROCESS: CURRENT INDUSTRIAL OPERATION (6)



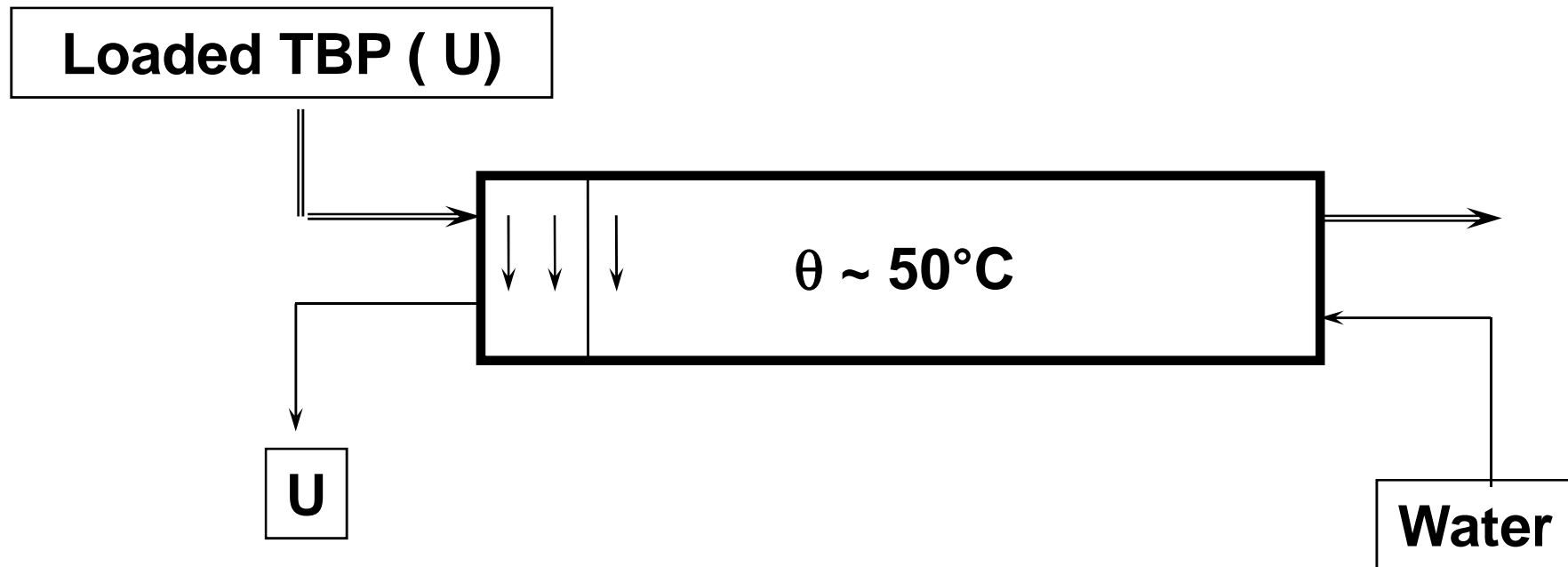
PUREX PROCESS: CURRENT INDUSTRIAL OPERATION (7)

SELECTIVE Pu STRIPPING: PRINCIPLE

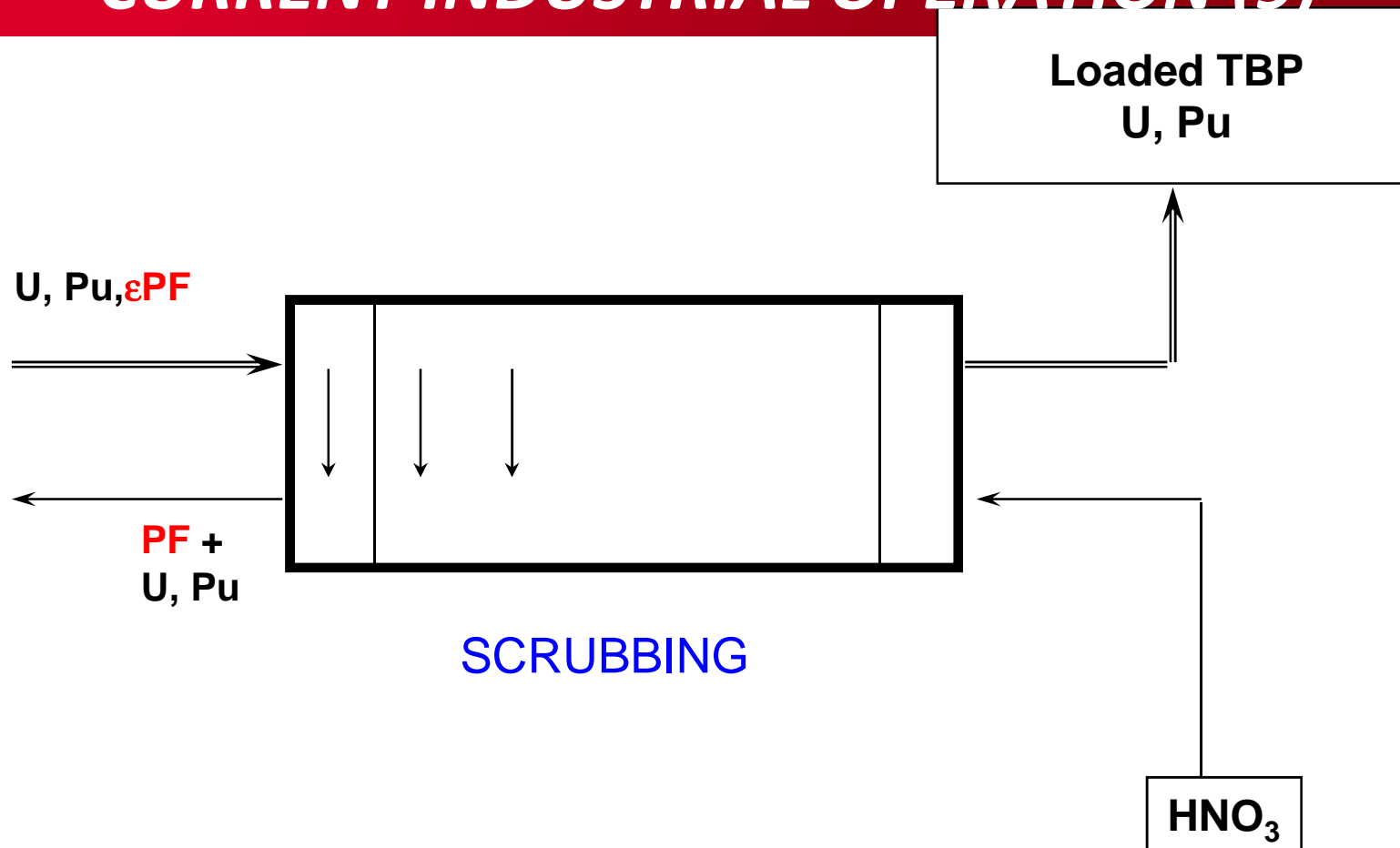


PUREX PROCESS: CURRENT INDUSTRIAL OPERATION (8)

URANIUM STRIPPING



PUREX PROCESS: CURRENT INDUSTRIAL OPERATION (9)

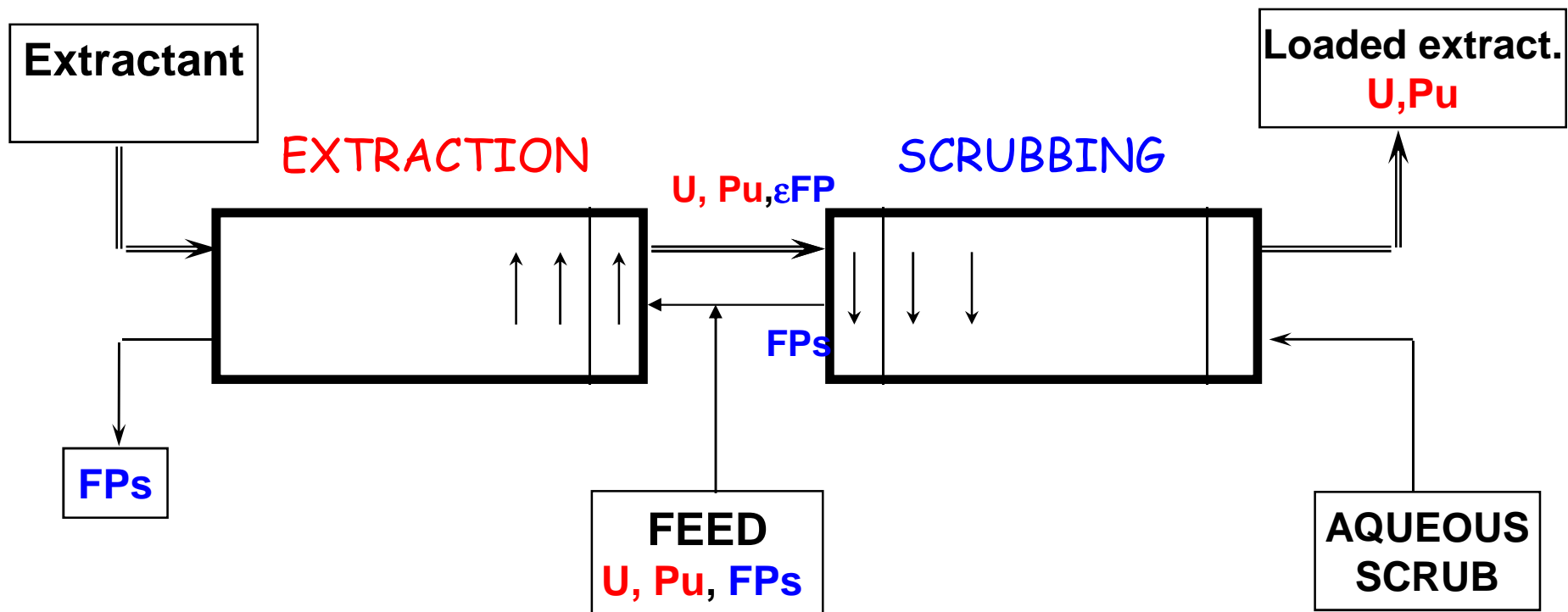


SCRUBBING OPERATIONS : PRINCIPLE

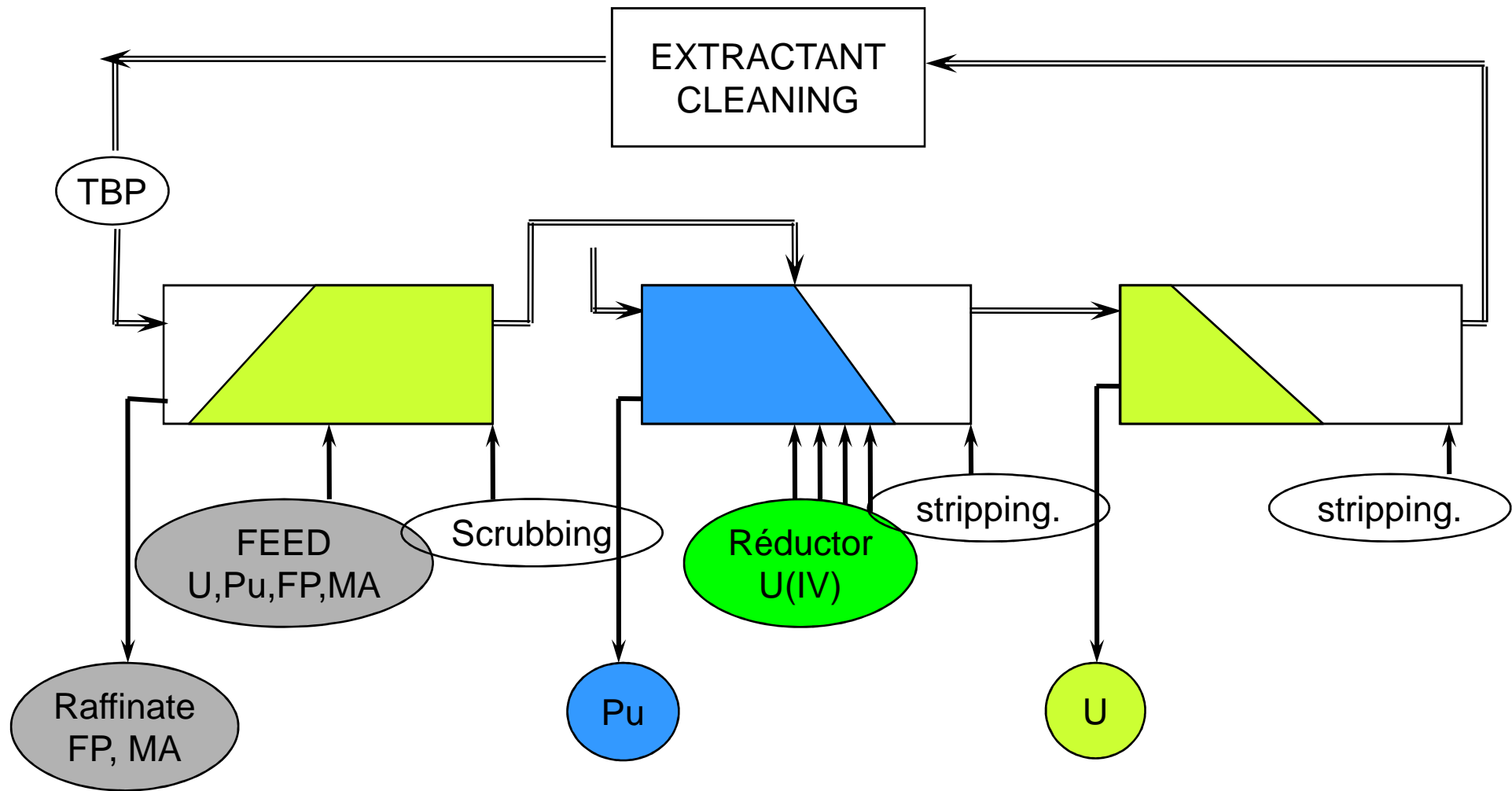


PUREX PROCESS: CURRENT INDUSTRIAL OPERATION (10)

SOLVENT EXTRACTION : High recovery yields, High purification yields !



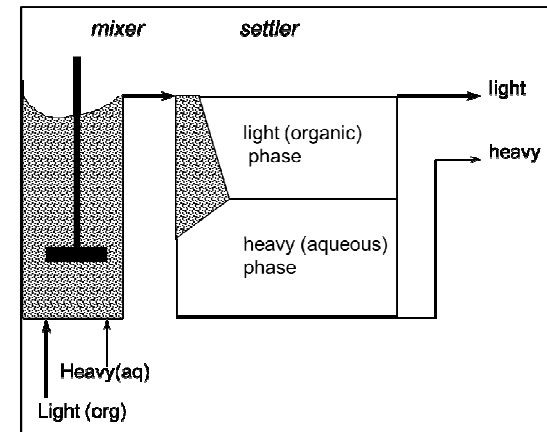
PUREX PROCESS: CURRENT INDUSTRIAL OPERATION (11)



1st CYCLE U-Pu WITH U/Pu PARTITIONING

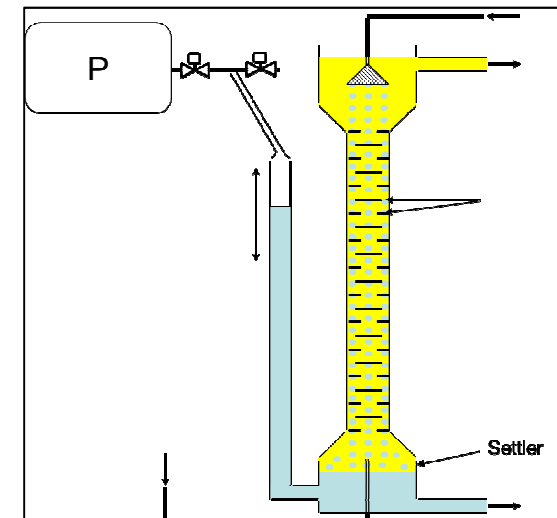
- **COMPARTIMENTED EXTRACTORS :**

- **MIXERS-SETTLERS**
- **CENTRIFUGAL CONTACTORS (MONO OU MULTI-ÉTAGES)**



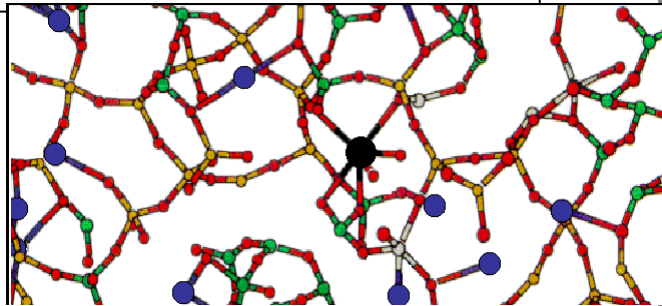
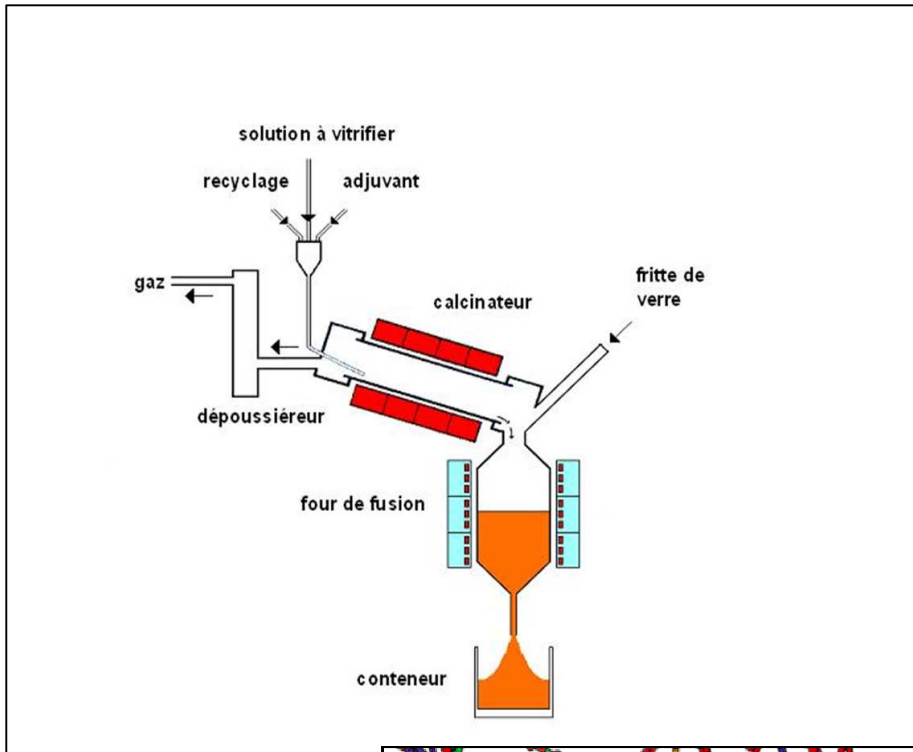
- **CONTINUOUS CONTACTORS :**

- **PULSED COLUMNS**



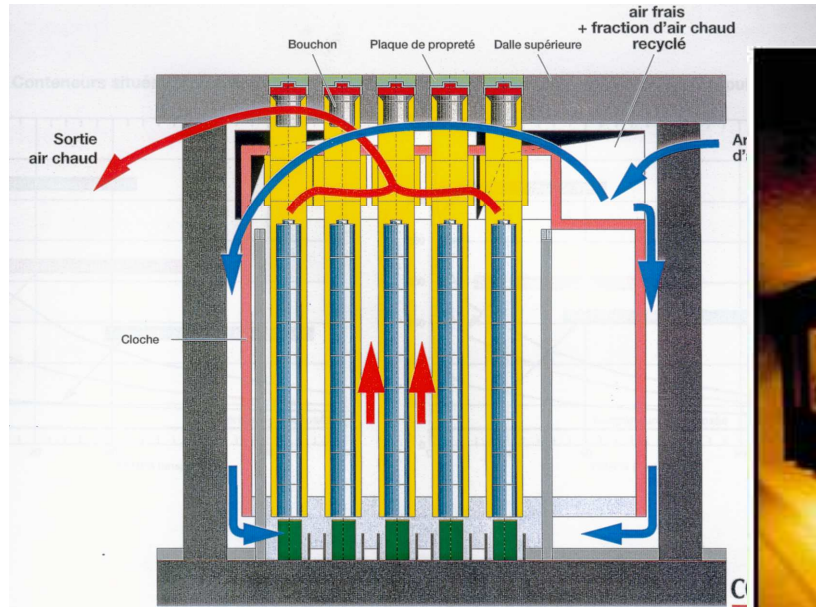
HLW CONDITIONING

GLASSES

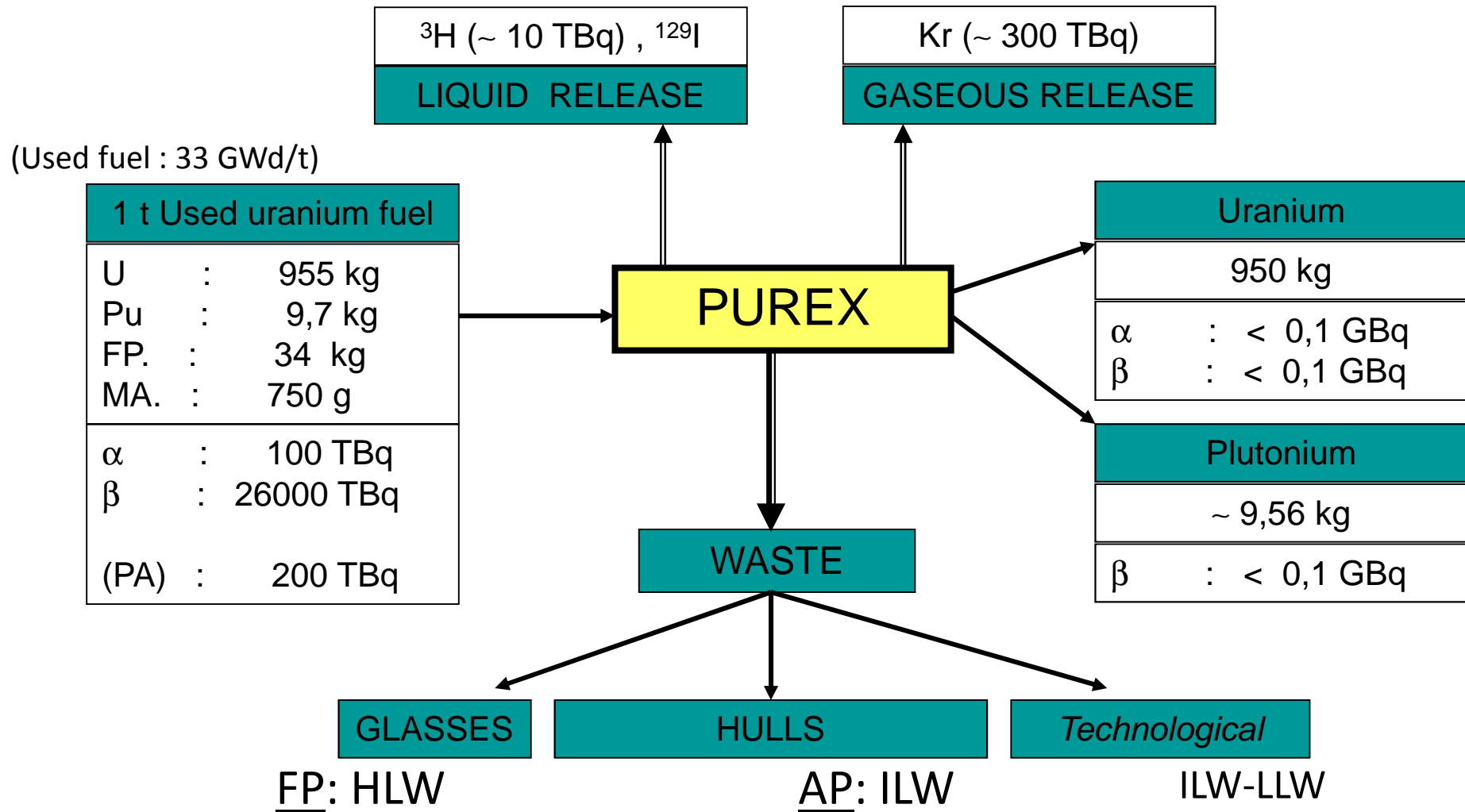


180 liters,
PF #15%, #2KW,
>15 000 TBq

GLASS CANISTERS INTERIM STORAGE (La HAGUE)



PUREX PROCESS AT LA HAGUE PLANT : RESULTS



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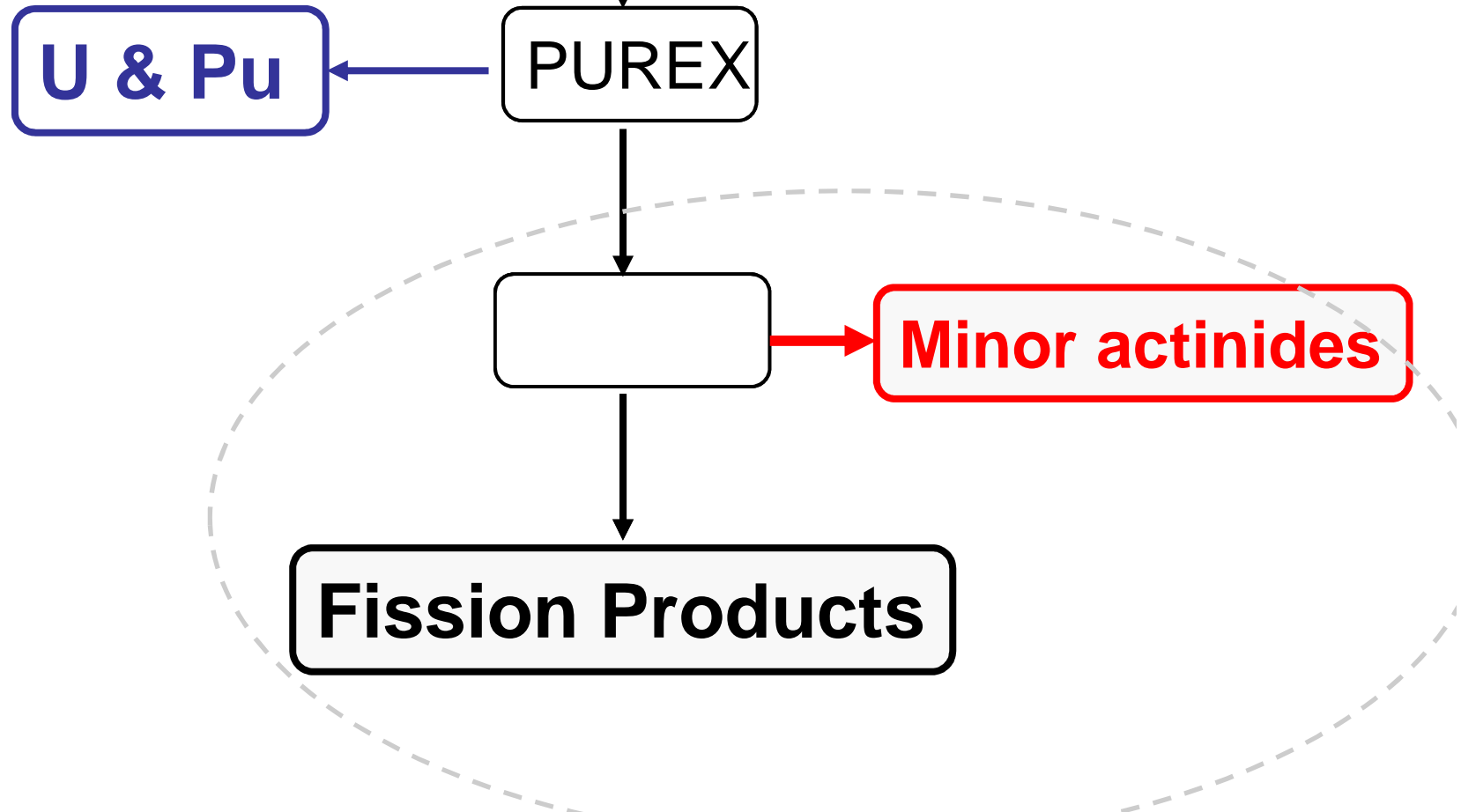
6 –3 MINOR ACTINIDE SEPARATION

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ADDITIONAL PROCESSES... ?



MINOR ACTINIDES SELECTIVE EXTRACTION after PUREX ?

- AMOUNTS (UOX "standard" fuel, 45 GWd/t) :

| | |
|--|---------|
| – NpO_2^+ / NpO_2^{2+} | 610 g/t |
| – Am^{3+} | 600 g/t |
| – Cm^{3+} | 90 g/t |

~ 1% used fuel content

- after PUREX:

- Highly **acidic** ($[\text{HNO}_3] \sim 3 \text{ mol.L}^{-1}$)
- Highly **radioactive** ($\text{FPs} \sim 5.10^{12} \text{ Bq.L}^{-1}$)
- Very diverse , and $[\text{Ln}] \sim 50 \times ([\text{Am}] + [\text{Cm}])$

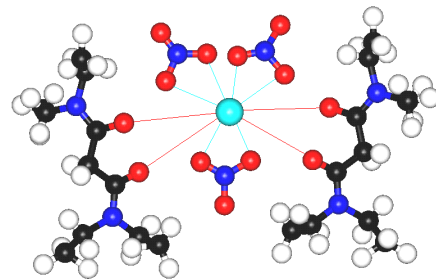
- CHEMICAL BEHAVIOR :

- Light actinides (Th to Pu) : *specific redox (PUREX basis)*
⇒ **Np recovery achievable by PUREX**
- Heavier actinides (from Am) : *behave like lanthanides*

DESIGN OF NEW EXTRACTANTS : MAIN CRITERIA

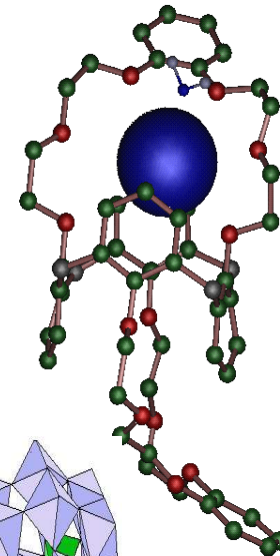
- Ability to separate

- *Affinity*
- *Selectivity*
- *Reversibility*



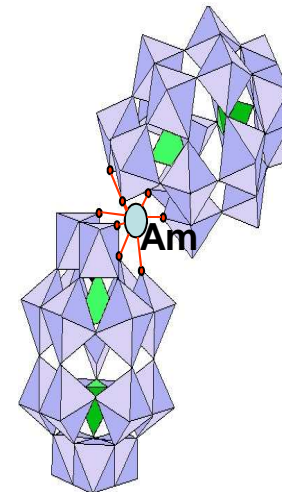
- Medium effects

- *Solubility*
- *Stability / hydrolysis, radiolysis*



- Industrialization

- *Kinetics*
- *Physical properties*
- *Cost*

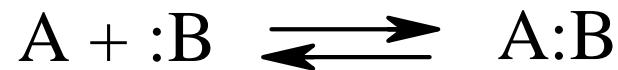


- Secondary waste minimization

- *Incinerability (C, H, O, N)*

HARD AND SOFT ACID BASE THEORY

A : Lewis acid, electron acceptor B : Lewis base, electron donor



PEARSON CLASSIFICATION : HARD / SOFT ACID/BASE (HSAB) THEORY

– Hard acid or base

small size

high charge density

hardly polarizable

Soft acid or base

large

low charge density

polarizable

« HARD ACIDS INTERACT RATHER WITH HARD BASES » (*ionic bounds*)

« SOFT ACIDS INTERACT RATHER WITH SOFT BASES » (*covalent bounds*)

Am³⁺ and Cm³⁺ SELECTIVE EXTRACTION : A KEY-ISSUE

- An³⁺ “hard cations” ⇒ “hard donor” ligands (O:)
(ionic bounds)

but poor selectivity vs Ln³⁺ !

| | | | | | | | | | | | | | | | | | |
|----|----|----|-----|-----|-----|-----|-----|-----|-----|----|----|----|----|----|----|----|----|
| 1 | | | | | | | | | | | | | | | | | 2 |
| H | | | | | | | | | | | | | | | | | He |
| 3 | 4 | | | | | | | | | | | 5 | 6 | 7 | 8 | 9 | 10 |
| Li | Be | | | | | | | | | | | B | C | N | O | F | Ne |
| 11 | 12 | | | | | | | | | | | 13 | 14 | 15 | 16 | 17 | 18 |
| Na | Mg | | | | | | | | | | | Al | Si | P | S | Cl | A |
| 19 | 20 | 21 | 22 | 23 | 24 | 25 | 26 | 27 | 28 | 29 | 30 | 31 | 32 | 33 | 34 | 35 | 36 |
| K | Ca | Sc | Ti | V | Cr | Mn | Fe | Co | Ni | Cu | Zn | Ga | Ge | As | Se | Br | Kr |
| 37 | 38 | 39 | 40 | 41 | 42 | 43 | 44 | 45 | 46 | 47 | 48 | 49 | 50 | 51 | 52 | 53 | 54 |
| Rb | Sr | Y | Zr | Nb | Mo | Tc | Ru | Rh | Pd | Ag | Cd | In | Sn | Sb | Te | I | Xe |
| 55 | 56 | 57 | 72 | 73 | 74 | 75 | 76 | 77 | 78 | 79 | 80 | 81 | 82 | 83 | 84 | 85 | 86 |
| Cs | Ba | Ln | Hf | Ta | W | Re | Os | Ir | Pt | Au | Hg | Tl | Pb | Bi | Po | At | Rn |
| 87 | 88 | 89 | 104 | 105 | 106 | 107 | 108 | 109 | 110 | | | | | | | | |
| Fr | Ra | An | Rf | Db | Sg | Bh | Hs | Mt | Uun | | | | | | | | |

| | | | | | | | | | | | | | | |
|-------------|----|----|----|----|----|----|----|----|----|----|-----|-----|-----|-----|
| LANTHANIDES | | | | | | | | | | | | | | |
| 57 | 58 | 59 | 60 | 61 | 62 | 63 | 64 | 65 | 66 | 67 | 68 | 69 | 70 | 71 |
| La | Ce | Pr | Nd | Pm | Sm | Eu | Gd | Tb | Dy | Ho | Er | Tm | Yb | Lu |
| 89 | 90 | 91 | 92 | 93 | 94 | 95 | 96 | 97 | 98 | 99 | 100 | 101 | 102 | 103 |
| Ac | Th | Pa | U | Np | Pu | Am | Cm | Bk | Cf | Es | Fm | Md | No | Lr |

ACTINIDE
FISSION PRODUCTS

ACTIVATION PRODUCTS
ACTIVATION AND FISSION PRODUCTS

Am³⁺ and Cm³⁺ SELECTIVE EXTRACTION : A KEY-ISSUE

- An³⁺ “hard cations” ⇒ “hard donor” ligands (O:)
(ionic bounds)

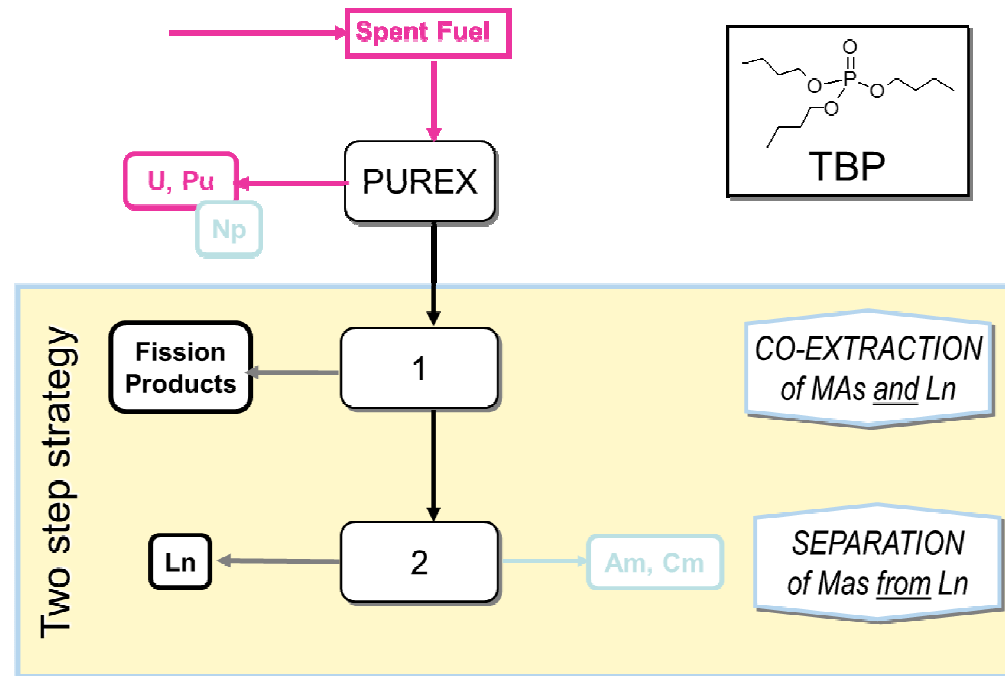
but poor selectivity vs Ln³⁺ !

- An³⁺ “softer” than Ln³⁺ ⇒ “soft donor” ligands (N, S)
(partly covalent bounds)

but poor selectivity vs other FPs !

vs H⁺!

Am³⁺ and Cm³⁺ SELECTIVE EXTRACTION : A TWO-STEP STRATEGY?



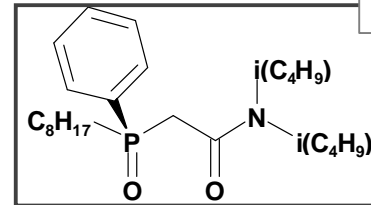
**1 - (An + Ln) co-extraction using hard-donor ligands
(e.g. CMPO, TRUEX process) (DIAMEX)**

**2 - An / Ln separation using soft-donor ligands
(e.g. DTPA, TALSPEAK process) (SANEX concept)**

An(III) and Ln(III) CO-EXTRACTION

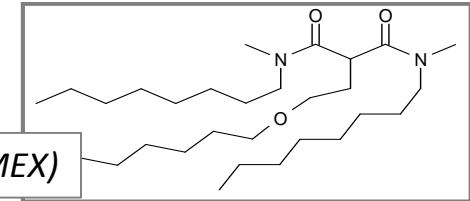
- BIDENDATE O –donnor LIGANDS

CMPO



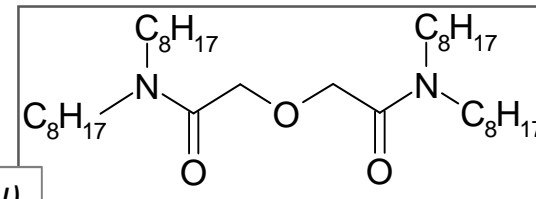
(USA – TRUEX)

(FRANCE – DIAMEX)



DMDOHEMA

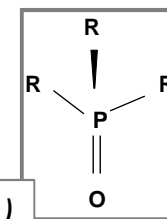
(JAPAN- EU)



TODGA

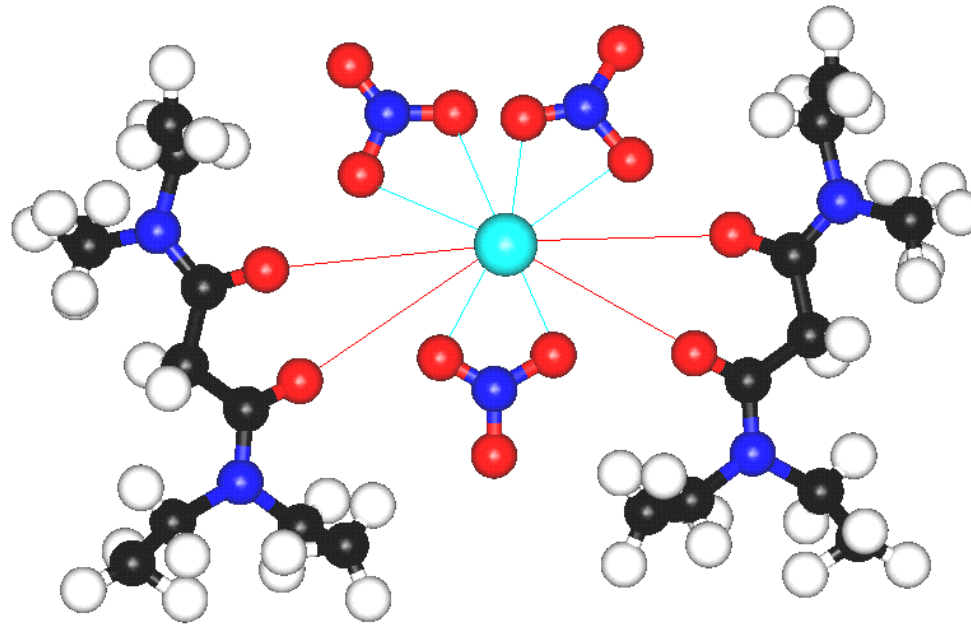
- *rather than* MONODENDATE ?

(CHINA)

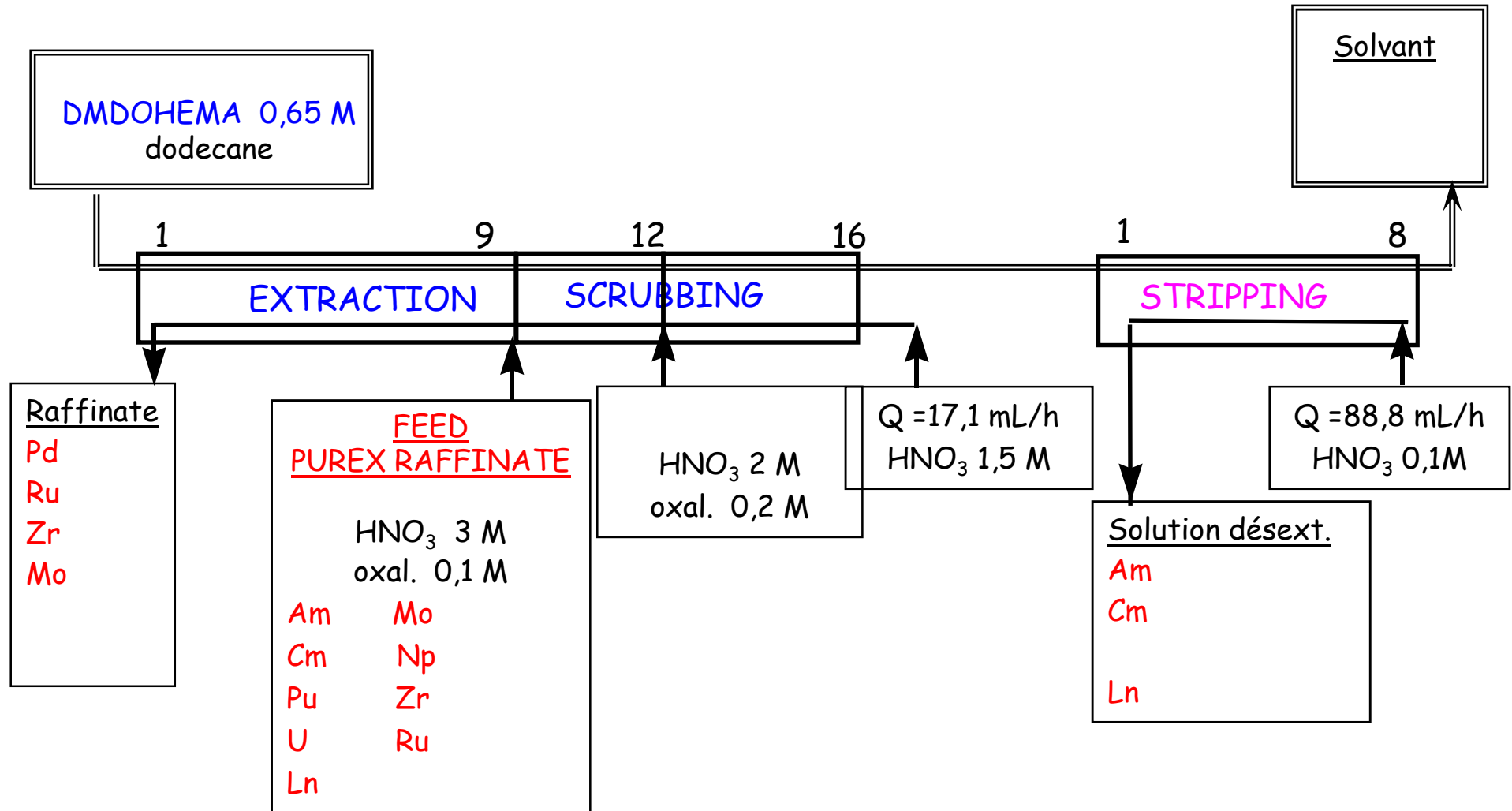


TRPO

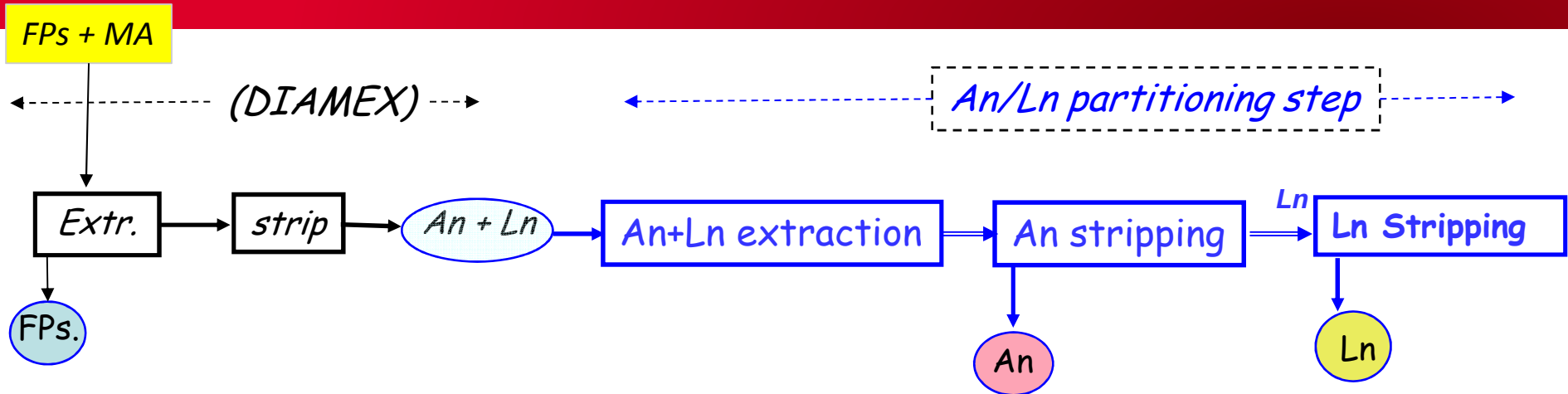
ACTINIDE EXTRACTION BY DIAMIDE



EXTRACTION USING DIAMIDE



An / Ln SEPARATION



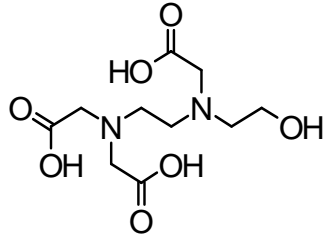
To be designed : An-selective complexant (« soft ligands)

A difficulty: efficient only if low acid content solutions...

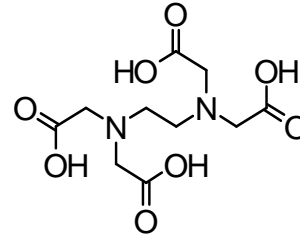
Options: adjust the nitric acid concentration in the stripping section
(but Ln must remain extracted!)

POLYAMINOPOLYCARBOXYLATES IN An / Ln SEPARATION

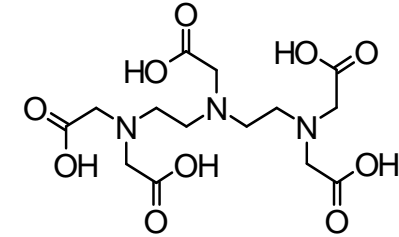
aqueous ligands, great selectivities for An vs Ln



HEDTA
N-(2-hydroxyethyl)
ethylenediaminetriacetic acid



EDTA
ethylenediaminetetracetic acid



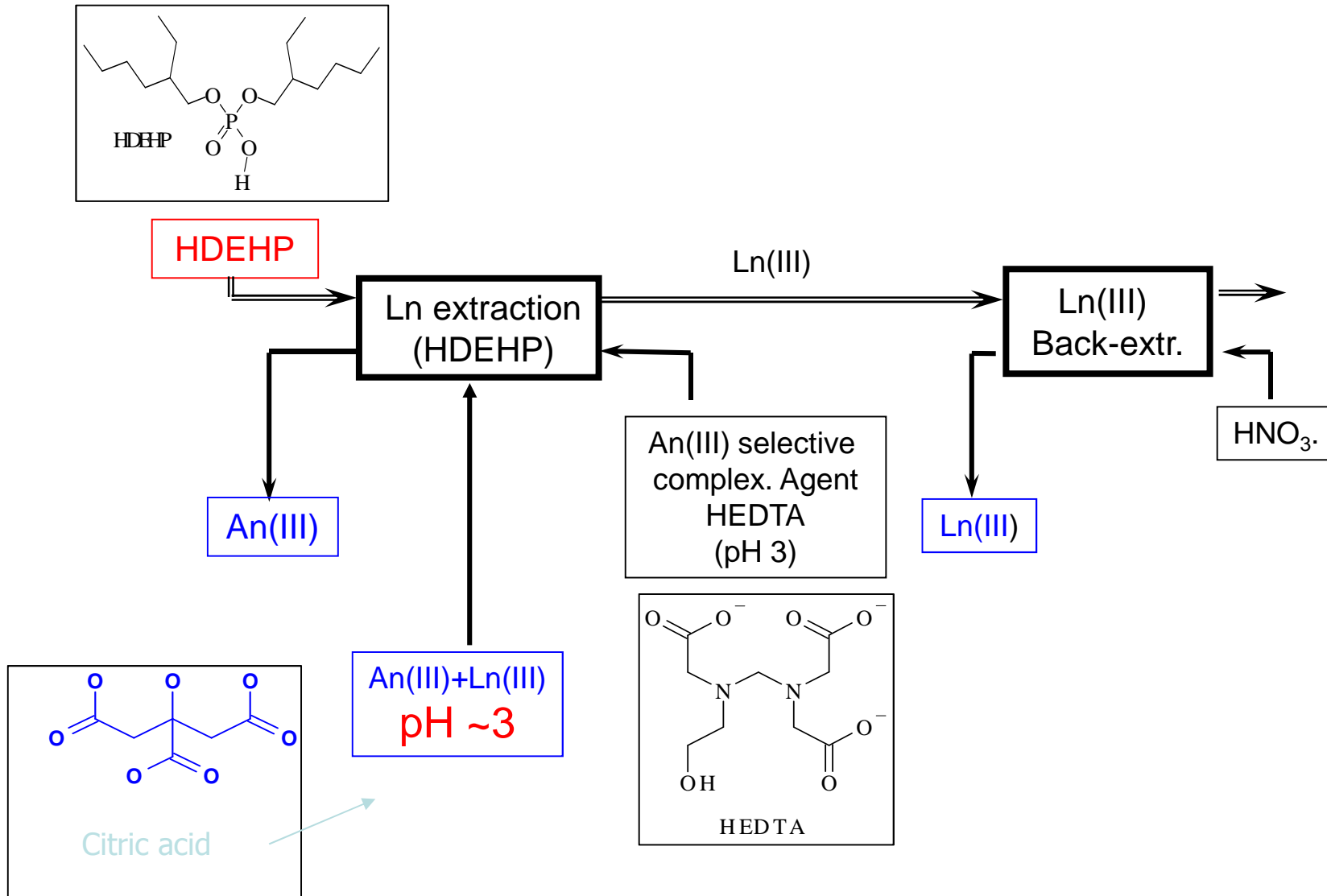
DTPA
diethylenetriaminepentacetic acid

- **Selective extraction of Ln**
 - TALSPEAK (US – 60s, DTPA)

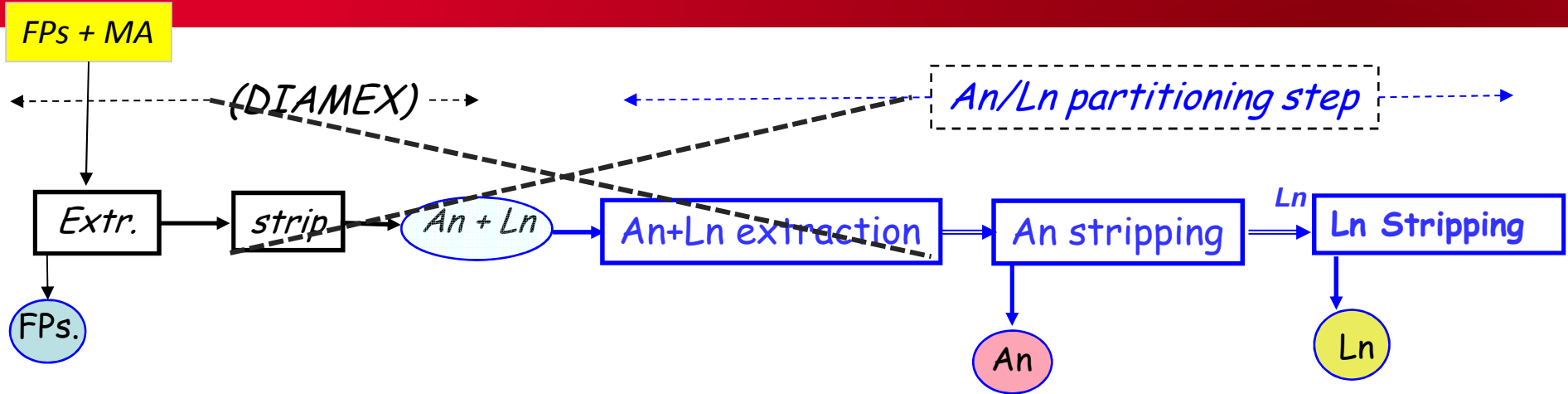
- **Selective back extraction of An**
 - Reverse TALSPEAK (variant of TALSPEAK)
 - SETFICS (Japon – 98, DTPA)
 - DIAMEX-SANEX (France – 99, HEDTA)

TALSPEAK PROCESS

(An/Ln separation step, after TRUEX)

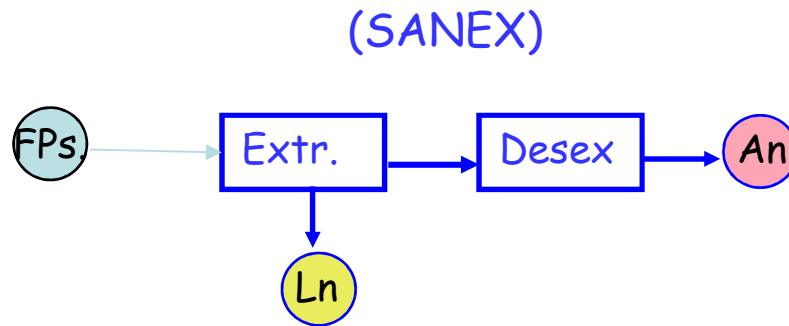


An / Ln SEPARATION



A single cycle after PUREX ?

An / Ln SEPARATION



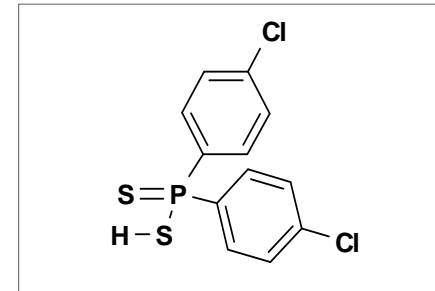
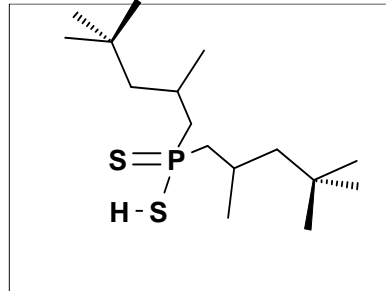
To be designed : An-selective extractant (« soft ligands » based)

A difficulty: efficient only if low acid content solutions...

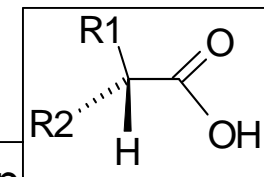
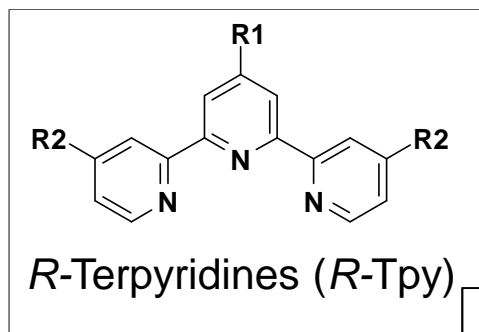
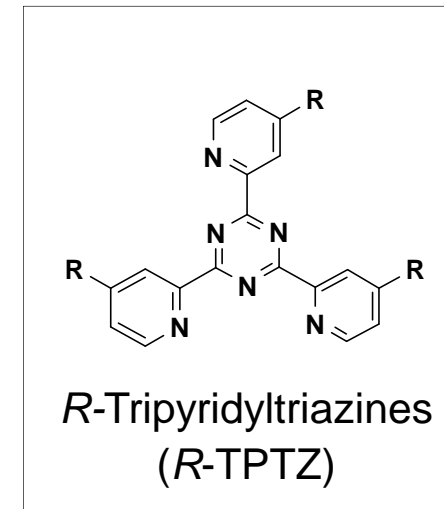
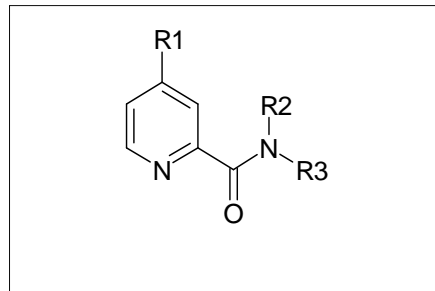
Options: adjust the nitric acid concentration in the feed ?
extractant design?

An / Ln SEPARATION : MANY MOLECULES TESTED

- Cation exchangers



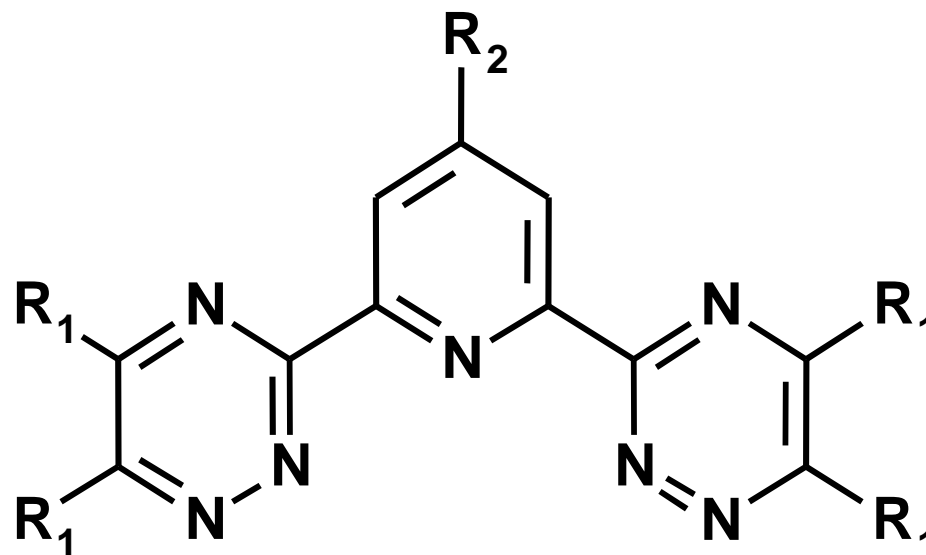
- Neutral extractants



with synergistic (lipophilic) counter ions!

Bis-Triazinyl-Pyridines (BTPs)

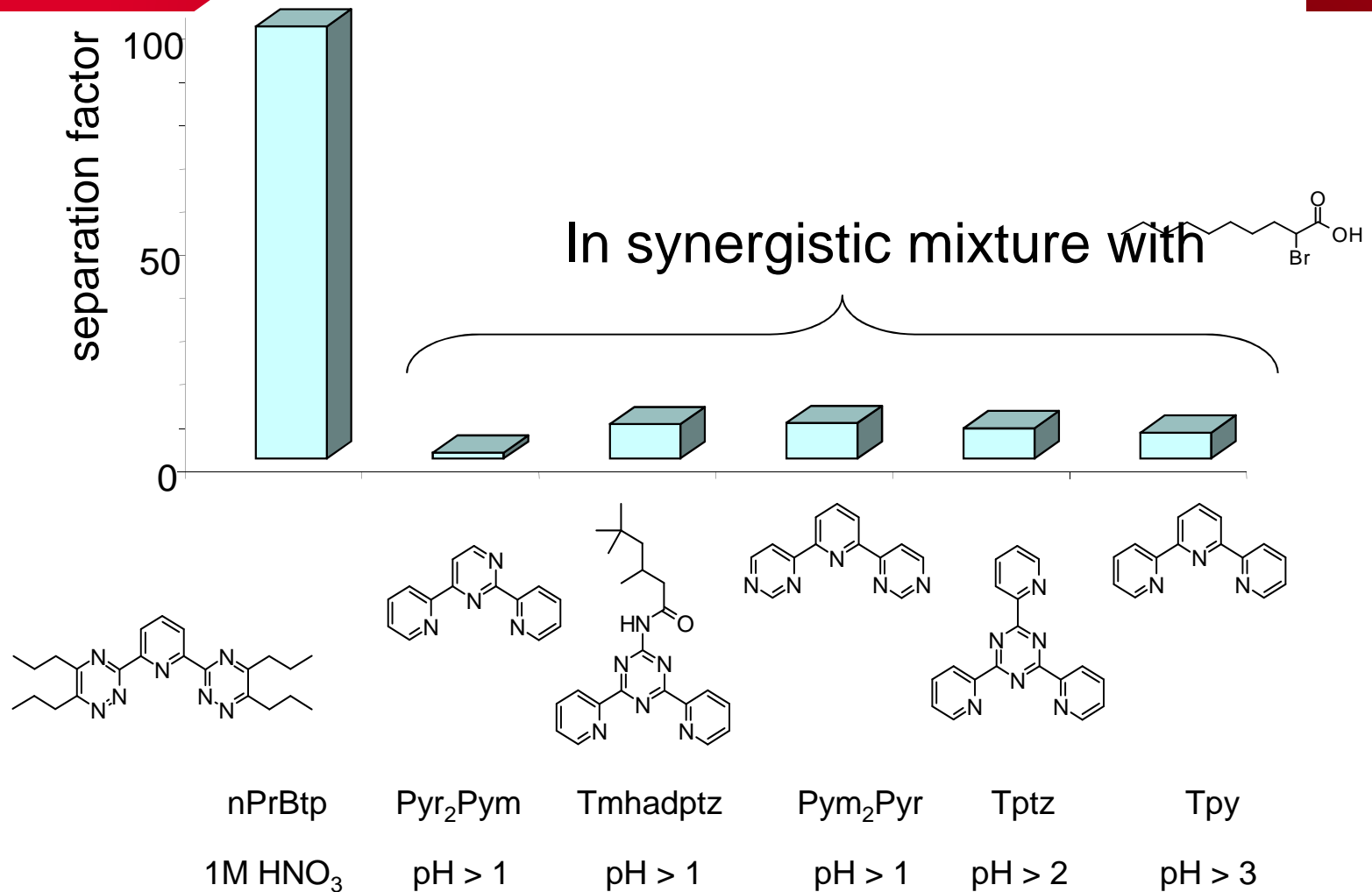
discovered by Dr. Z. KOLARIK in 1998 (NEWPART project)



$R_1 = \text{H, Me, } n\text{-Propyl, } i\text{-Pr, } n\text{-Butyl, } i\text{-Bu, (1-Me)Pr, } neo\text{-Pe, } i\text{-Pe, } \phi, \phi\text{-OMe, Phen, Pyr}$

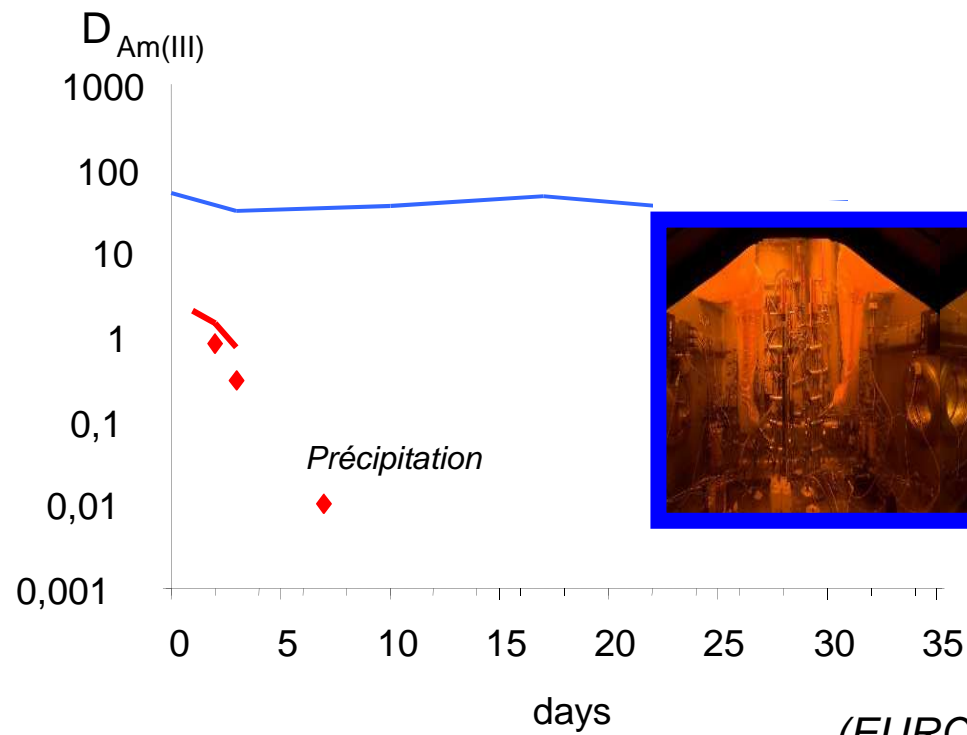
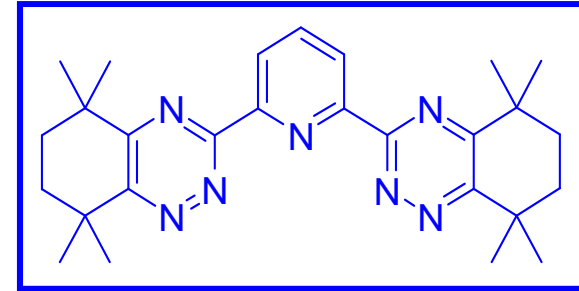
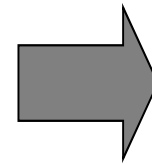
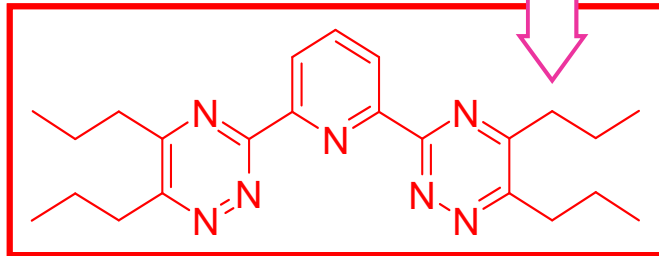
$R_2 = \text{H, } i\text{-Nonyl}$

An(III) / Ln(III) SEPARATION BY POLYAZINES



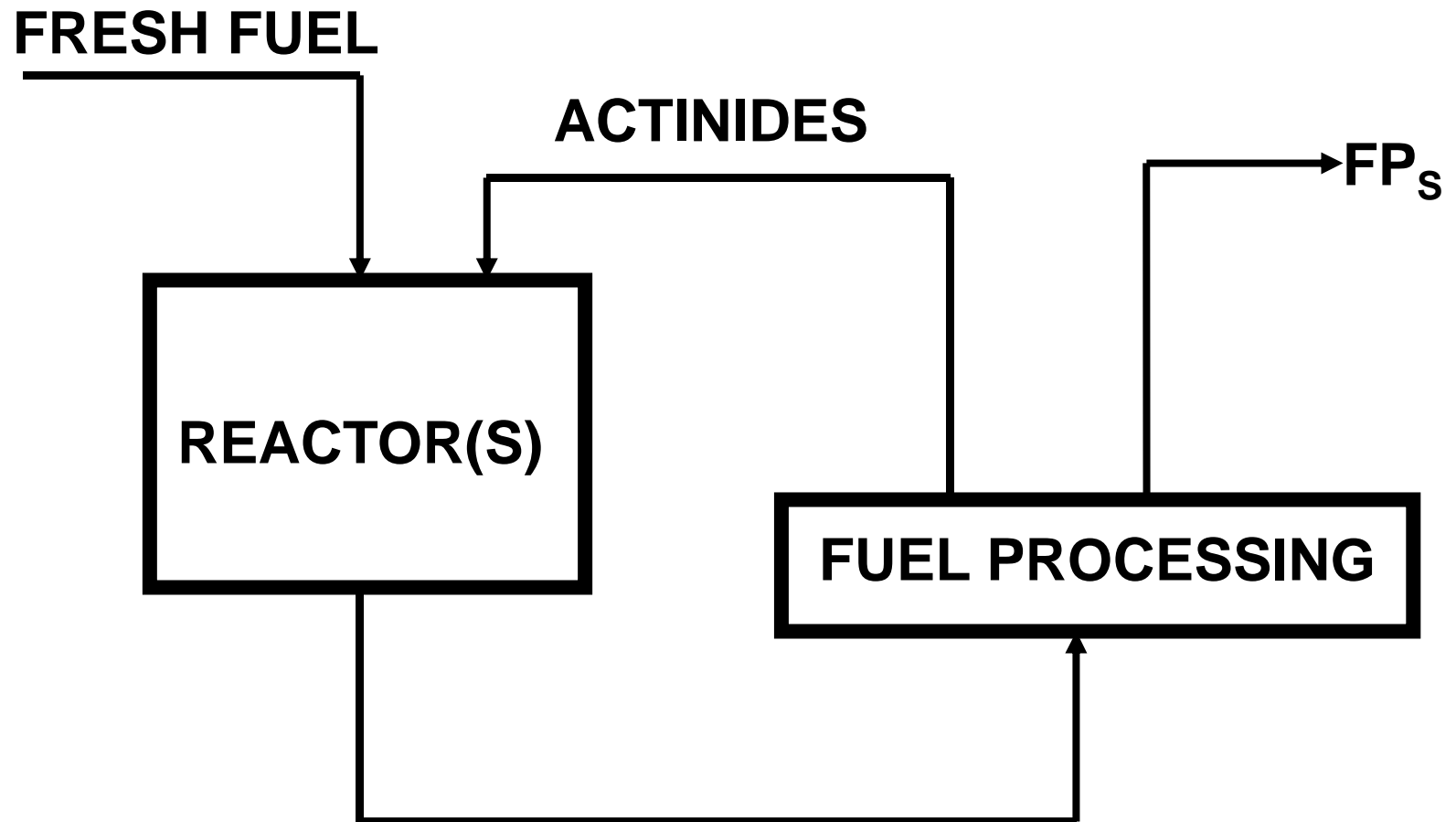
EXTRACTANT'S STABILITY

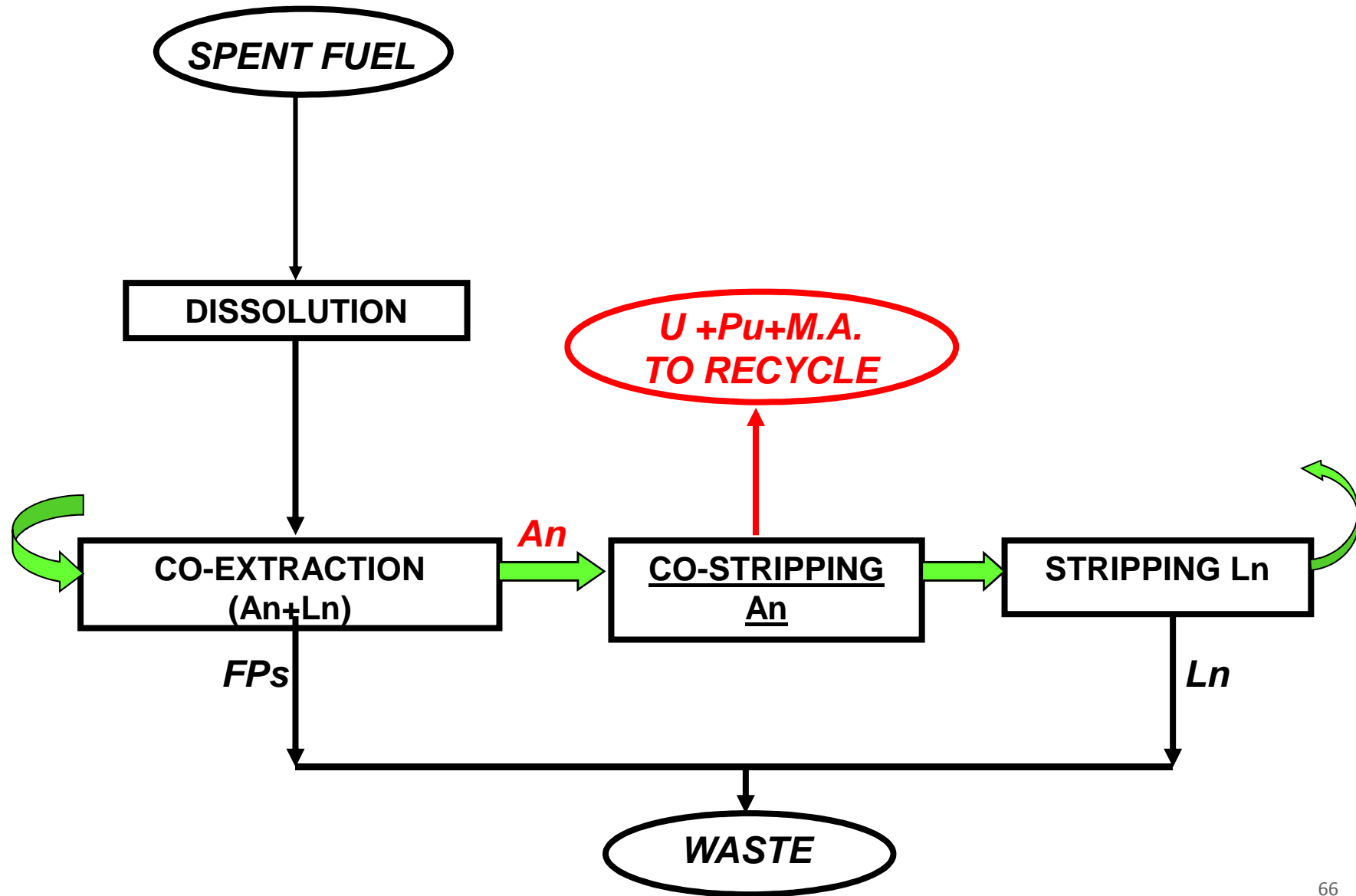
weakness



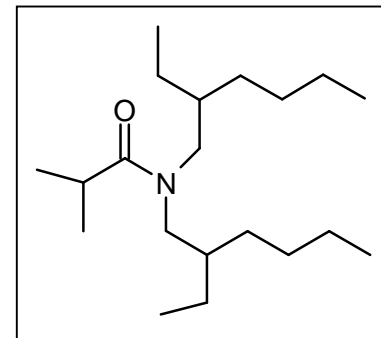
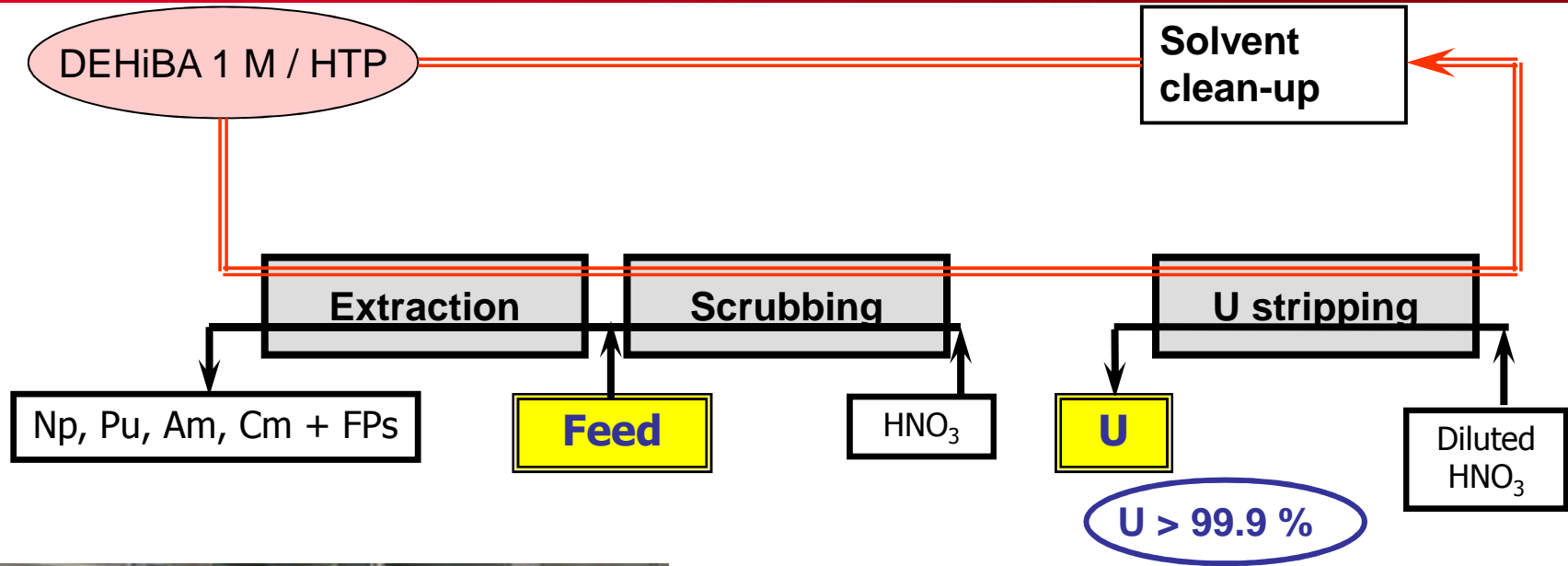
(EUROPART project, PCRD 5th)

ALL-ACTINIDE RECYCLE ?



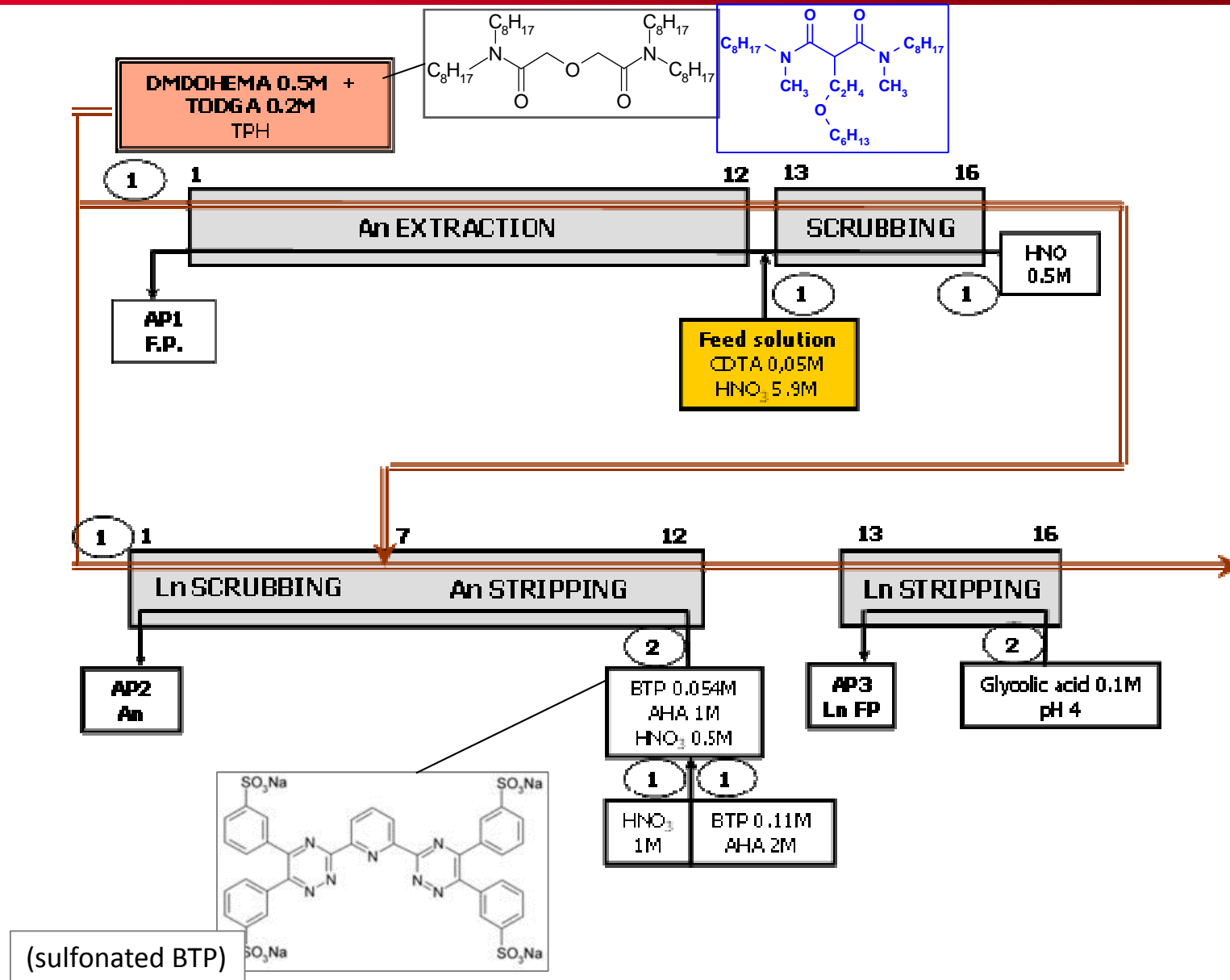


GANEX 1st step (*U extraction*) (Monoamide)



« EUROGANEX »

(SACCESS & GENIORS EU projects)



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NUCLEAR FUEL CYCLES

6-4 AND FOR OTHER FUELS....

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Joliot Curie School, St Pierre d'Oleron, 22-27 September, 2019

WHY CARBIDE ? (HN density,thermal conductivity,...)

CARBIDE FUEL : U (Pu) C , but also UC₂, U₂C₃,...

CARBIDE FUEL DISSOLUTION:

- $UC + 6 HNO_3 \Rightarrow UO_2(NO_3)_2 + CO_2 + 3 H_2O + 3 NO + NO_2$
- $PuC + 8 HNO_3 \Rightarrow Pu(NO_3)_4 + CO_2 + 4 H_2O + 2 NO + 2 NO_2$

(rather high kinetics)

SOME ISSUES FOR « PUREX-CARBIDE »:

**-Organic compounds (carboxylic acids,...) at the dissolution step
[and behavior in downstream process steps]**

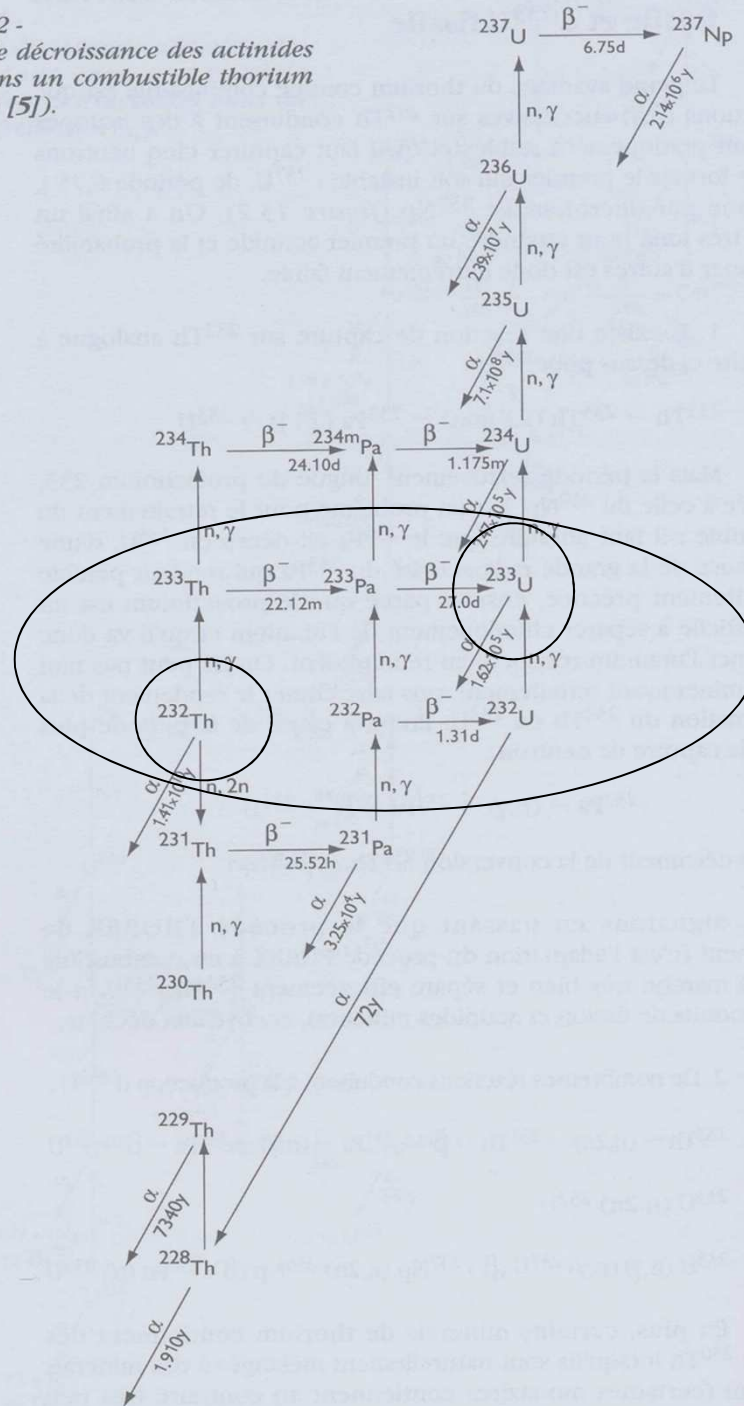
-Pyrophoricity risk ?

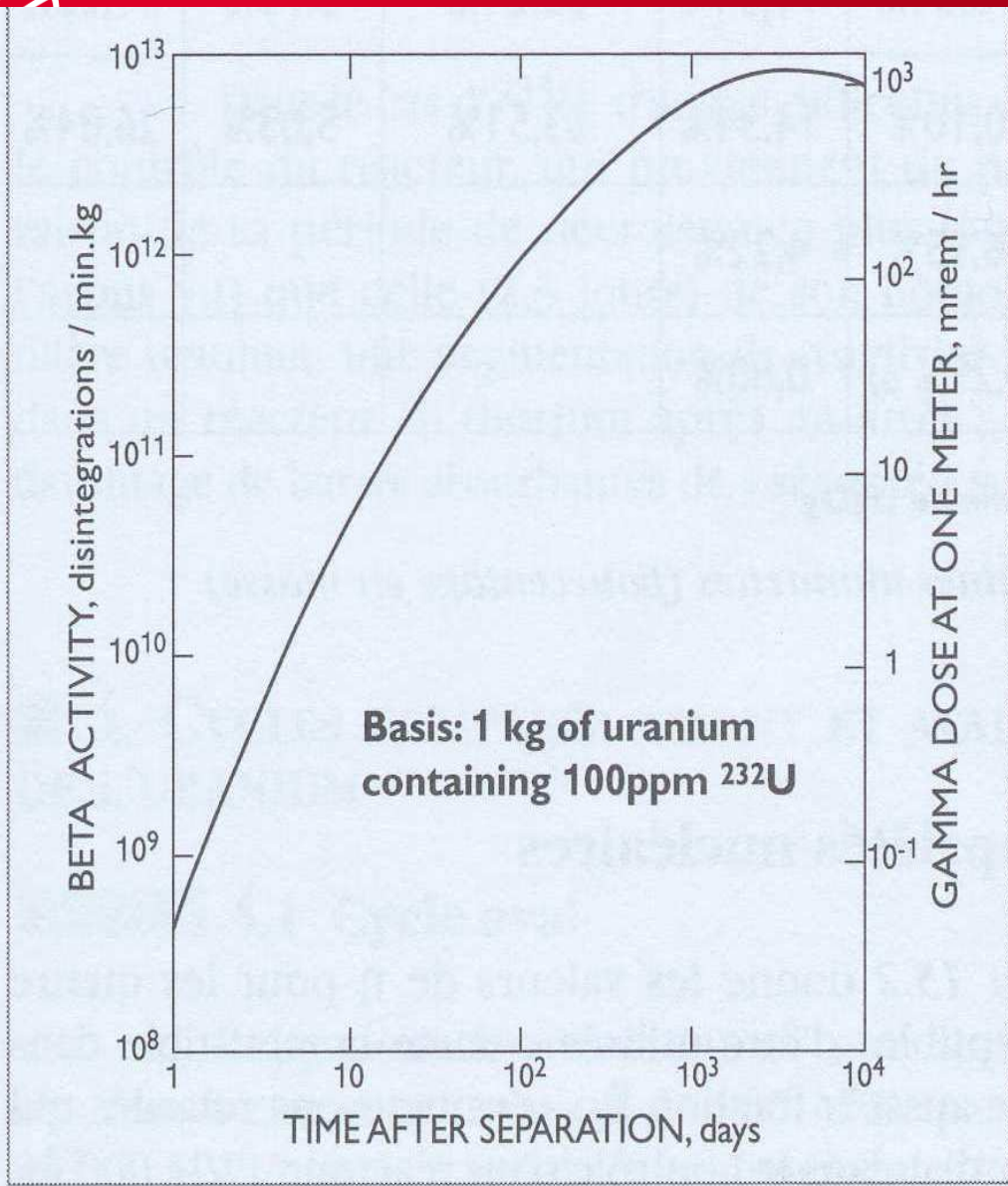
- ^{233}U fissile, but not natural
- ^{232}Th reserves: 3 times uranium ones
- ^{232}Th « fertile » : ^{233}Pa , decay: ^{233}U ;
- ^{233}U : high multiplication factor
- Necessity of prior cycle with another fissile nuclei to fertilize ^{232}Th

| Nuclide | σ_a (barns) | σ_f (barns) |
|-------------------|--------------------|--------------------|
| ^{232}Th | 7.40 | 0 |
| ^{231}Pa | 227 | 0.01 |
| ^{233}Pa | 41.5 | 0 |
| ^{232}U | 149.7 | 77.1 |
| ^{233}U | 571.1 | 525.2 |

| Nuclide | Fast neutrons | Thermal neutrons |
|---------------|---------------|------------------|
| Uranium 233 | 2.3 | 2.3 |
| Uranium 235 | 1.9 | 2.1 |
| Plutonium 239 | 2.3 | 2.1 |

Figure 15.2 -
Chaînes de décroissance des actinides
formés dans un combustible thorium
(référence [5]).





THOREX process specific issues:
 -*dissolution*
 -*no redox for separation*

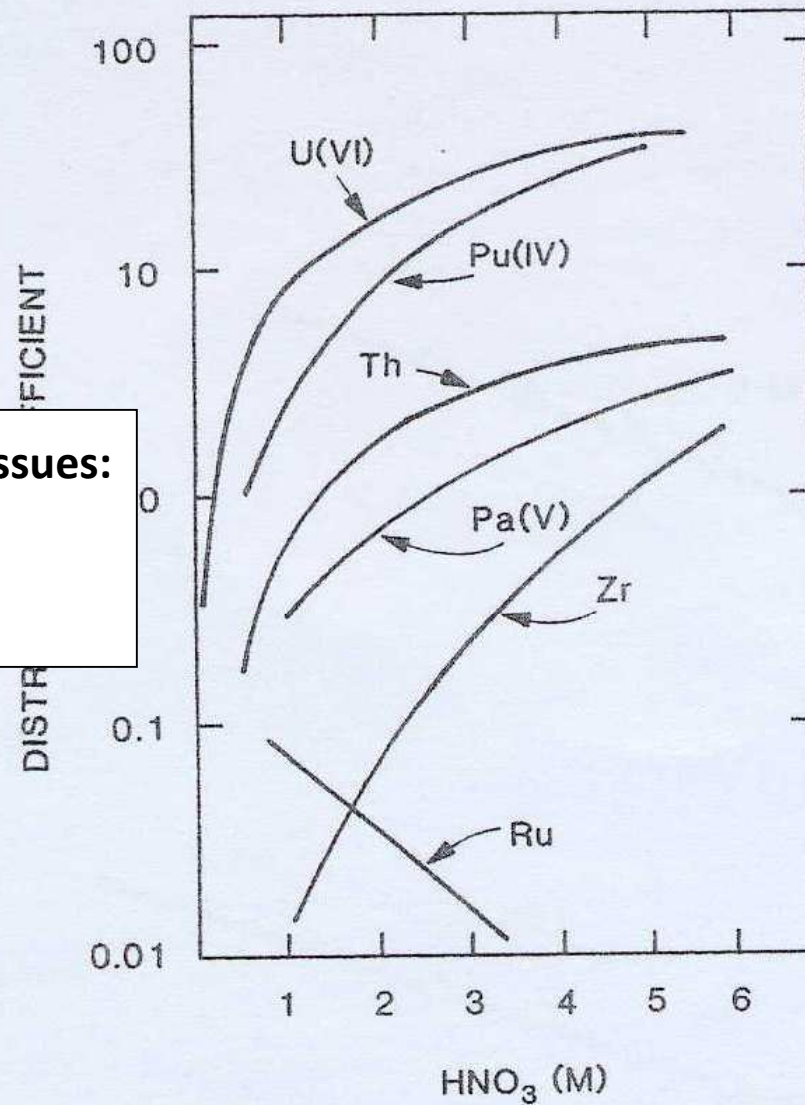


FIGURE 5. Distribution of thorium and key actinides and fission products between 30% TBP solutions and aqueous HNO₃. (Drawn from data presented in References 6 and 17 to 19.)

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NUCLEAR FUEL CYCLES

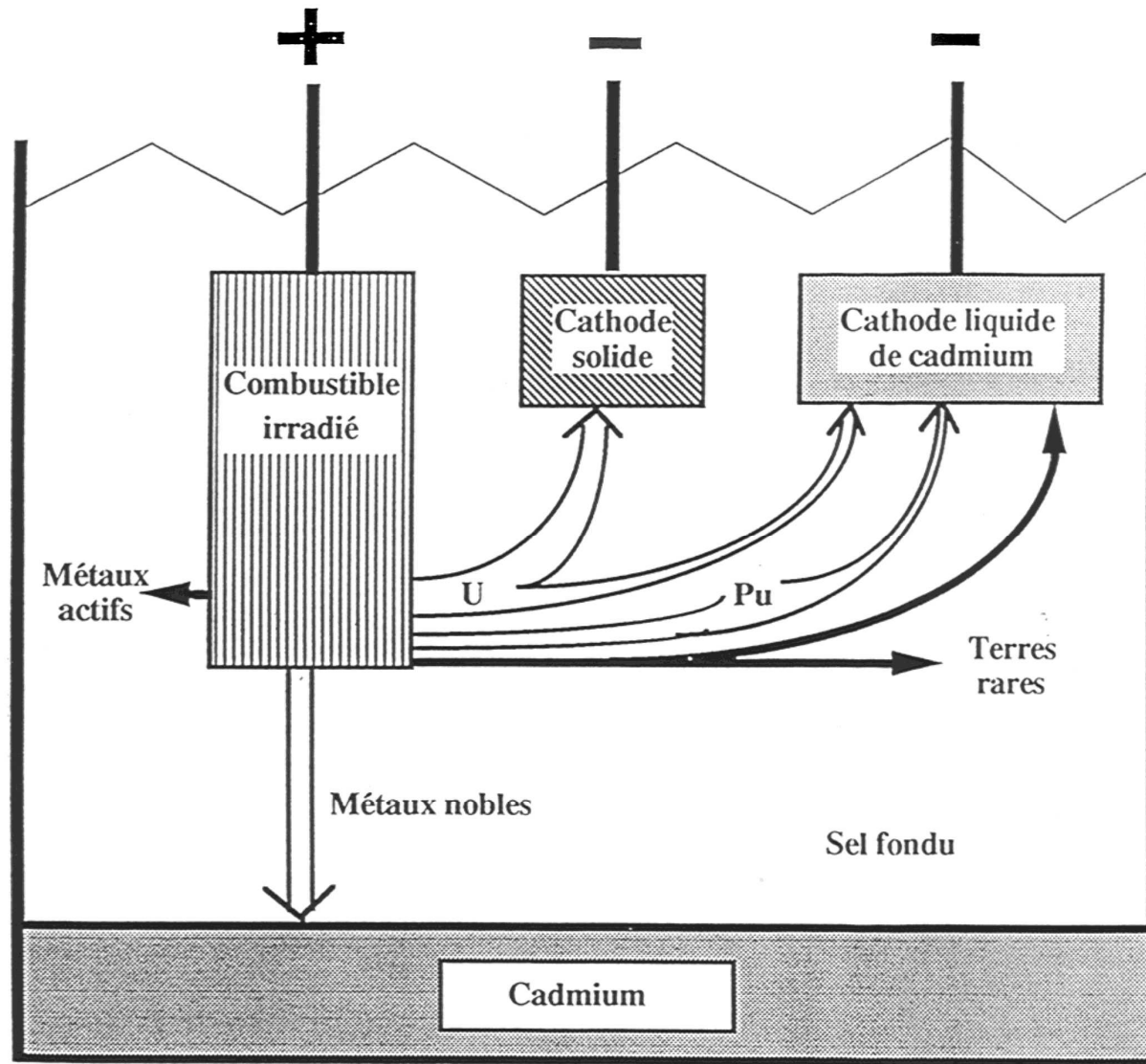
6 –5 PYRO-PROCESSES

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Joliot Curie School, St Pierre d'Oleron, 22-27 September, 2019

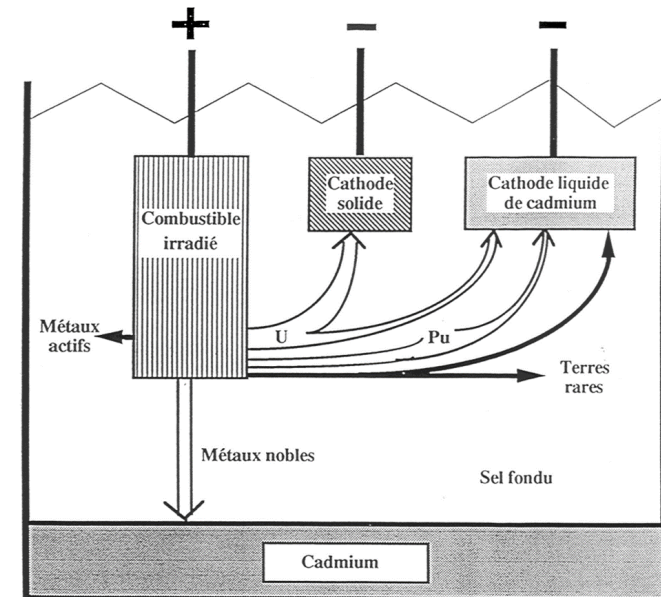
PYRO-PROCESSES : ARGONNE PILOT-SCALE EXPERIENCE



- **The challenge** : to distribute selectively the actinides (An) and the fission products (FPs) :
 - *between a salt, and a metal (solid or liquid)*
 - *electrochemical or extraction processes*
- **Scientific challenge comes down** :
 - *to select salt phase for which the gap between free energies of formation of An and FP compounds is the largest one*
 - *to select metallic phase having the highest affinity for An compounds*
- **from lab-scale to demonstrative experiments on genuine spent fuel samples**

PRESUMED ADVANTAGES :

- (1) *Low radiolytical effects*
- (2) *Solvating properties*
- (3) *Compactness (in the principle)*
- (4) *Extended range of electro-activity*
- (5) *No neutron moderator (criticality risk)*
- (6) *Molten salts reactors (obviously...)*
- (7) ...



UNCERTAINTIES, POTENTIAL DRAWBACKS :

- (1) *Recovery & purification yields ?*
- (2) *Minor actinide recovery ?*
- (3) *Operating the technology (corrosion, technological waste,...)*
- ⁸⁰ (4) *Oxide fuels?*